

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

JULY 1985

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Becky Brown and Carol Kronich, telephone number 309-654-2241, extensions 127 and 193.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

July 1-7

Unit One began the month of July increasing load to full power. On July 1, at 2115 hours, load reached 810 MWe. On July 2, at 1652 hours, the unit was placed on Economic Generation Control (EGC). EGC was tripped on July 6, at 0012 hours, and load was dropped to 700 MWe for Turbine surveillances. At 0605 hours load began an increase to 730 MWe and, the unit was placed on EGC at 0642 hours.

July 8-15

On July 8, at 0730 hours, EGC was tripped and the unit began a load increase to full power. At 1300 hours, load reached 812 MWe and held steady. On July 9, at 0030 hours, load began a decrease to 730 MWe so that condenser flow could be reversed. At 0140 hours load began an increase to 800 MWe. At 0305 hours load was dropped to 750 MWe in preparation for EGC. At 0340 hours the unit was placed on EGC. On July 10, at 220^c hours, load began a drop to minimum power as per the Load Dispatcher. Minimum power was reached on July 11 at 0105 hours. At 0452 hours load began a normal increase to full power. On July 12, at 0325 hours, load held steady at 812 MWe.

July 16-24

On July 16, at 1730 hours, load was dropped to 790 MWe in preparation for EGC. The unit was placed on EGC at 1748 hours. On July 21, at 0030 hours, EGC was tripped and load began a drop to 550 MWe for a rod pattern exchange. Five hundred and fifty MWe was reached at 0310 hours, and load was increased normally. On July 22, at 1720 hours, load was decreased by 30 MWe to 700 MWe for rod adjustments.

On July 23, at 0835 hours, load began a drop to 550 MWe for rod pattern adjustments due to thermal limit problems. Load reached 550 MWe at 1800 hours, and began a 5 MWe/hour increase at 1930 hours.

On July 24, at 1645 hours, load was dropped to 550 MWe for rod pattern maneuvers due to thermal limit problems. At 2100 hours load was increased 100 MWe/hour to 650 MWe and then increased 5 MWe/hour.

July 25-31

On July 25, at 1120 hours, a normal orderly shutdown commenced at 25 MWe/hour due to Technical Specifications, because LPCI became inoperable. At 1412 hours the shutdown was terminated at 650 MWe, and a normal load increase to full power began. On July 27, at 0815 hours, load reached 812 MWe and remained steady. On July 28, at 0015 hours, load was dropped to 700 MWe for Turbine surveillances. At 0135 hours load began an increase to 808 MWe and held steady for the remainder of July.

B. Unit Two

July 1-7

Unit Two began the month holding load at 809 MWe. On July 2, at 0835 hours, load was dropped to 770 MWe in preparation for Economic Generation Control (EGC). At 0924 hours, the unit was placed on EGC. On July 7, at 0050 hours, EGC was tripped and load was decreased to 700 MWe for Turbine surveillances. At 0305 hours load was increased to 740 MWe and at 0340 hours the unit was placed on EGC.

July 8-10

On July 8, at 0725 hours, EGC was tripped and load began a normal increase to full power. Eight hundred and ten MWe was reached at 0822 hours on July 9; at 0315 hours load was decreased to 750 MWe for EGC operation. At 0355 hours, 750 MWe was reached and the unit was placed on EGC. At 0611 hours, EGC was tripped and load was increased to full power. On July 10, at 0405 hours, load was decreased to 700 MWe as per the Load Dispatcher. At 0505 hours the load decrease was stopped and the unit began an increase to full power.

July 11-31

On July 11, at 0100 hours, load began a decrease to 650 MWe as per the Load Dispatcher. Six hundred and fifty MWe was reached at 0230 hours. At 0545 hours, load began a normal increase to full power. Full power was reached at 1410 hours and held steady. On July 20, at 0015 hours, load was decreased to 600 MWe for Turbine surveillances and was held for main transformer problems and tests. On July 22, at 0840 hours, load began an increase to full power. On July 27, at 0030 hours, load was dropped to 700 MWe for Turbine surveillances. At 0600 hours load began a normal increase to full power. Full power was reached at 1545 hours and was held steady. On July 31, at 0030 hours, load was dropped to 790 MWe and the unit was placed on EGC.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

On February 25, 1985, the NRC issued Amendment 85 to License DPR-30. This amendment establishes an increase in the maximum allowable Linear Heat Generation Rate (LHGR) for the End of Cycle 7 Barrier Fuel Ramp Test.

On May 30, 1985, the NRC issued Amendments 86 and 87 to License DPR-30. Amendment 86 establishes the implementation of Analog Trip Systems in the Reactor Low Water Level, Reactor Water Level, HPCI High Steam Flow, and HPCI Steamline Low Pressure Instrumentation. It also establishes the installation of level switches and related instrumentation in the Scram Discharge Volumes. This includes changes to the pertinent Reactor Protection System Instrumentation Requirements, Logic Systems Functional Tests, and Minimum Calibration Frequencies Tables. The amendment also shows the new Maximum Average Planar Linear Heat Generation Rate (MAPLHGR) versus Planar Average Exposure Curve for the new Unit Two core.

Amendment 87 establishes Hafnium metal as an allowable material of control rods.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
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There were no Safety Related Work Requests closed out for Unit One for the reporting period.

UNIT 2 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q42478	85-13	Unit 2 Diesel Generator	Governor compensa- ting mechanism was out of adjustment due to normal operational wear.	Diesel Generator ran 30 seconds, then tripped out on over- speed. Unit 1/2 Diesel Generator out of service. Declared GSEP.	Correctly adjusted the governor compensating mechanism.
Q42952		2B Main Steam Line Radiation Monitor (Recalibrated)	Instrument set- point drift.		
Q43519		2-1001-18A RHR Pump Minimum Flow Valve	Unknown at this time. See Work Request Q43885.	Valve was cycling open and closed with no pumps on.	Valve operator was found separated from valve body, and was continuously operating. This gave an erroneous open and close signal. For final action see Work Request Q43885; August 1985 Monthly Report.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1. and 6.6.B.2. of the Technical Specifications.

UNIT ONE

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-09	7-25-85	Low Pressure Coolant Injection Inoperable; Required Shutdown
85-10	7-28-85	Reactor Core Isolation Cooling Inoperable Due to Failed Overspeed Meter

UNIT TWO

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-16	7-29-85	Standby Gas Treatment Auto-Start Due to Fuel Pool Monitor Spike

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE AUGUST 6 1985

COMPLETED BY CAROL L. KRONICH

TELEPHONE (309) 654-2241 X19

OPERATING STATUS

0000 070185

1. Reporting period: 2400 073185 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

This Month Yr. to Date Cumulative

5. Number of hours reactor was critical	<u>744.0</u>	<u>4838.0</u>	<u>93160.4</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>3421.9</u>
7. Hours generator on line	<u>744.0</u>	<u>4780.4</u>	<u>89815.7</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>909.2</u>
9. Gross thermal energy generated (MWH)	<u>1724887</u>	<u>11206254</u>	<u>186952644</u>
10. Gross electrical energy generated (MWH)	<u>561727</u>	<u>3693568</u>	<u>60471019</u>
11. Net electrical energy generated (MWH)	<u>537014</u>	<u>3536102</u>	<u>56491804</u>
12. Reactor service factor	<u>100.0</u>	<u>95.1</u>	<u>88.4</u>
13. Reactor availability factor	<u>100.0</u>	<u>95.1</u>	<u>83.3</u>
14. Unit service factor	<u>100.0</u>	<u>94.0</u>	<u>77.5</u>
15. Unit availability factor	<u>100.0</u>	<u>94.0</u>	<u>78.3</u>
16. Unit capacity factor (Using MDC)	<u>93.9</u>	<u>90.4</u>	<u>63.4</u>
17. Unit capacity factor (Using Des. MWe)	<u>91.5</u>	<u>88.1</u>	<u>61.8</u>
18. Unit forced outage rate	<u>0.0</u>	<u>6.0</u>	<u>6.1</u>

19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):

20. If shutdown at end of report period, estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE AUGUST 6 1985

COMPLETED BY CAROL L. KRONICH

TELEPHONE (309) 654-2241x19

OPERATING STATUS

0000 070185

1. Reporting period: 2400 073185 Gross hours in reporting period: 744

2. Currently authorized power level (MWt): 2511 Max. Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted (if any) (MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr. to Date	Cumulative
5. Number of hours reactor was critical	<u>744.0</u>	<u>2975.8</u>	<u>87881.8</u>
6. Reactor reserve shutdown hours	<u>0.0</u>	<u>0.0</u>	<u>2985.8</u>
7. Hours generator on line	<u>744.0</u>	<u>2903.9</u>	<u>84953.1</u>
8. Unit reserve shutdown hours.	<u>0.0</u>	<u>0.0</u>	<u>702.9</u>
9. Gross thermal energy generated (MWH)	<u>1780205</u>	<u>6649052</u>	<u>178168119</u>
10. Gross electrical energy generated (MWH)	<u>575235</u>	<u>2161789</u>	<u>56815200</u>
11. Net electrical energy generated (MWH)	<u>550535</u>	<u>2065040</u>	<u>53383025</u>
12. Reactor service factor	<u>100.0</u>	<u>58.5</u>	<u>76.4</u>
13. Reactor availability factor	<u>100.0</u>	<u>58.5</u>	<u>79.0</u>
14. Unit service factor	<u>100.0</u>	<u>57.1</u>	<u>73.9</u>
15. Unit availability factor	<u>100.0</u>	<u>57.1</u>	<u>74.5</u>
16. Unit capacity factor (Using MDC)	<u>96.2</u>	<u>52.8</u>	<u>60.4</u>
17. Unit capacity factor (Using Des. MWe)	<u>93.8</u>	<u>51.5</u>	<u>58.8</u>
18. Unit forced outage rate	<u>0.0</u>	<u>6.2</u>	<u>8.3</u>
19. Shutdowns scheduled over next 6 months (Type, Date, and Duration of each):			
20. If shutdown at end of report period, estimated date of startup			<u>NA</u>

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254

UNIT ONE

DATE AUGUST 6 1985

COMPLETED BY CAROL L. KRONICH

TELEPHONE (309) 654-2241X19

MONTH July 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>764.3</u>
2.	<u>741.3</u>
3.	<u>741.8</u>
4.	<u>731.8</u>
5.	<u>731.6</u>
6.	<u>716.5</u>
7.	<u>728.9</u>
8.	<u>759.3</u>
9.	<u>728.1</u>
10.	<u>725.1</u>
11.	<u>617.6</u>
12.	<u>773.5</u>
13.	<u>765.0</u>
14.	<u>759.2</u>
15.	<u>764.6</u>
16.	<u>759.9</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>714.3</u>
18.	<u>730.8</u>
19.	<u>734.5</u>
20.	<u>725.8</u>
21.	<u>578.1</u>
22.	<u>654.9</u>
23.	<u>607.7</u>
24.	<u>588.2</u>
25.	<u>651.7</u>
26.	<u>719.2</u>
27.	<u>772.7</u>
28.	<u>762.5</u>
29.	<u>774.2</u>
30.	<u>775.0</u>
31.	<u>780.0</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE AUGUST 6 1985

COMPLETED BY CAROL L. KRONICH

TELEPHONE (309) 654-2241X19

MONTH July 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>800.5</u>
2.	<u>718.2</u>
3.	<u>723.5</u>
4.	<u>716.2</u>
5.	<u>735.0</u>
6.	<u>733.7</u>
7.	<u>732.8</u>
8.	<u>754.1</u>
9.	<u>754.9</u>
10.	<u>773.5</u>
11.	<u>712.5</u>
12.	<u>763.2</u>
13.	<u>764.0</u>
14.	<u>761.5</u>
15.	<u>762.2</u>
16.	<u>761.7</u>

17.	<u>764.8</u>
18.	<u>769.3</u>
19.	<u>761.0</u>
20.	<u>573.0</u>
21.	<u>580.2</u>
22.	<u>663.6</u>
23.	<u>748.7</u>
24.	<u>776.4</u>
25.	<u>752.9</u>
26.	<u>767.4</u>
27.	<u>736.0</u>
28.	<u>765.7</u>
29.	<u>766.6</u>
30.	<u>769.7</u>
31.	<u>778.1</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13

Revision 6

August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY C. Kronich

DATE August 6, 1985

REPORT MONTH JULY 1985

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
85-34	850706	S	0.0	H	5		HA	TURBIN	Reduced load to 700 MWe for Turbine surveillances
85-35	850709	S	0.0	H	5		XX	ZZZZZZ	Reduced load to 750 MWe to place unit on Economic Generation Control (EGC)
85-36	850710	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load to minimum per Load Dispatcher
85-37	850716	S	0.0	H	5		XX	ZZZZZZ	Reduced load to 790 MWe to place unit on Economic Generation Control (EGC)
85-38	850721	S	0.0	H	5		RC	CONROD	Reduced load to 550 MWe for Control Rod pattern adjustment
85-39	850723	S	0.0	H	5		RC	FUELXX	Reduced load to 550 MWe for rod pattern adjustment due to thermal limit problems
85-40	850724	S	0.0	H	5		RC	FUELXX	Reduced load to 550 MWe for rod pattern adjustment due to thermal limit problems

APPROVED

AUG 16 1982

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONSQTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY C. Kronich

DATE August 6, 1985

REPORT MONTH JULY 1985

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
85-41	850725	F	0.0	D	5	85-09	ZZ	ZZZZZZ	Commenced orderly shutdown due to LPCI inoperability (terminated shutdown-- 2.87 hours)
85-42	850728	S	0.0	H	5		HA	TURBIN	Reduced load to 700 MWe for Turbine surveillances

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AUG 16 1982

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APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13

Revision 6

August 1982

DOCKET NO. 050-265UNIT NAME Quad-Cities Unit 2COMPLETED BY C. KronichDATE August 6, 1985REPORT MONTH JULY 1985TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
85-26	850702	S	0.0	H	5		XX	ZZZZZZ	Reduced load to 770 MWe to place unit on Economic Generation Control (EGC)
85-27	850707	S	0.0	H	5		HA	TURBIN	Reduced load to 700 MWe for Turbine surveillances
85-28	850709	S	0.0	H	5		XX	ZZZZZZ	Reduced load to 750 MWe to place unit on Economic Generation Control (EGC)
85-29	850710	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load per Load Dispatcher
85-30	850711	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load per Load Dispatcher
85-31	850720	S	0.0	H	5		HA	TURBIN	Reduced load to 600 MWe for Turbine surveillances, and remained at 600 MWe for main transformer problems and tests
85-32	850727	S	0.0	H	5		HA	TURBIN	Reduced load to 700 MWe for Turbine surveillances

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AUG 16 1982

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. MAIN STEAM RELIEF VALVE OPERATIONS

There were no Main Steam Relief Valve Operations for the reporting period.

B. CONTROL ROD DRIVE SCRAM TIMING DATA FOR UNITS ONE AND TWO

There was no Control Rod Drive Scram Timing Data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

*

1. Unit: Q1 Reload: 7 Cycle: 8
2. Scheduled date for next refueling shutdown: 1-2-86
3. Scheduled date for restart following refueling: 4-2-86
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
NOT AS YET DETERMINED.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
DECEMBER 19, 1985; IF LICENSING ACTION REQUIRED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
NONE PLANNED AT PRESENT TIME.
7. The number of fuel assemblies.
- a. Number of assemblies in core: 724
- b. Number of assemblies in spent fuel pool: 2340
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
- a. Licensed storage capacity for spent fuel: 3657
- b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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APR 20 1978

Q. C. O. S. R.

QUAD-CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 1
March 1978

- *
1. Unit: Q2 Reload: 7 Cycle: 8
2. Scheduled date for next refueling shutdown: 10-6-86
3. Scheduled date for restart following refueling: 12-22-86
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:
Yes. A routine MAPLHGR amendment has been approved as a preparatory change to allow a 10 CFR 50.59 review.
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
Not Applicable.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
None planned at present time.
7. The number of fuel assemblies.
a. Number of assemblies in core: 724
b. Number of assemblies in spent fuel pool: 176
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
a. Licensed storage capacity for spent fuel: 3897
b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

APPROVED

APR 20 1978

Q. C. O. S. R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center



Commonwealth Edison

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TKT-85-42

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Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, DC 20555

Attention Document Control Desk

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of July, 1985.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

T. K. Tanlyn for
T. K. Tanlyn
Services Superintendent

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Enclosure

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