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**POWERING
MICHIGAN'S PROGRESS**

Big Rock Point Nuclear Plant, 10269 US-31 North, Charlevoix, MI 49720

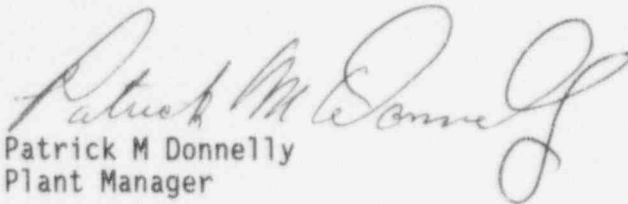
Patrick M Donnelly
Plant Manager

January 31, 1997

Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

**DOCKET 50-155 - LICENSE DPR-6 - BIG ROCK POINT PLANT - LICENSEE EVENT REPORT
97-001: TEST NOT PERFORMED IN ACCORDANCE WITH APPENDIX J TO 10 CFR PART 50.**

Licensee Event Report 97-001, **TEST NOT PERFORMED IN ACCORDANCE WITH APPENDIX J
TO 10 CFR PART 50**, is attached. This event is reportable to the Nuclear
Regulatory Commission in accordance with 10 CFR 50.73(a)(2)(i) - Any operation
prohibited by the plant's technical specifications



Patrick M Donnelly
Plant Manager

CC: Administrator, Region III, USNRC
NRC Resident Inspector - Big Rock Point

ATTACHMENT

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Patrick M Donnelly (Signed)

Patrick M Donnelly
Plant Manager

CC: Administrator, Region III, USNRC
NRC Resident Inspector - Big Rock Point

ATTACHMENT

LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY
INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE
INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY.
FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND
RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY
COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK
REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET,
WASHINGTON, DC 20503

FACILITY NAME (1)

BIG ROCK POINT NUCLEAR PLANT

DOCKET NUMBER (2)

50-155

PAGE (3)

1 OF 3

TITLE (4)

Test Not Performed in Accordance With Appendix J to 10 CFR Part 50.

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
1	2	97	97	001	00	1	31	97	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)		0%	20.2201(b)		20.2203(a)(2)(v)		X	50.73(a)(2)(i)		50.73(a)(2)(viii)
			20.2203(a)(1)		20.2203(a)(3)(i)			50.73(a)(2)(ii)		50.73(a)(2)(x)
			20.2203(a)(2)(i)		20.2203(a)(3)(ii)			50.73(a)(2)(iii)		73.71
			20.2203(a)(2)(ii)		20.2203(a)(4)			50.73(a)(2)(iv)		OTHER
			20.2203(a)(2)(iii)		50.36(c)(1)			50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A
			20.2203(a)(2)(iv)		50.36(c)(2)			50.73(a)(2)(vii)		

LICENSEE CONTACT FOR THIS LER (12)

NAME

Michael D. Bourassa, Licensing Supervisor

TELEPHONE NUMBER (INCLUDE AREA CODE)

1-616-547-8244

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRPDS
D	BD	NA	NA	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE).	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

Big Rock Point Technical Specification 3.7(d) requires that "containment sphere integrated leakage rate tests (Type A, B, and C) shall be conducted in accordance with the requirements specified in Appendix J to 10 CFR Part 50, as modified by approved exemptions".

On January 2, 1997, at 1948 hours, plant procedure TV-02, Containment Integrated Leak Rate Test (ILRT), was initiated. The purpose of this procedure is to pressurize the containment sphere through penetration H-80 and its associated containment isolation valve VCI-1, then measure leakage after 8 hours. When the air compressors were valved in to supply air into the containment sphere, and the test could not be performed as expected, the engineers determined that a blind flange had been installed between system piping flanges. The blind flange is believed to have been in this location since 1992, the date of the last ILRT. This condition is contrary to Big Rock Point Technical Specification 3.7(d) because any Local Leak Rate Tests (LLRTs) conducted on this penetration since 1992 were not performed in accordance with Appendix J criteria.

The root cause concluded that TV-02 did not provide adequate guidance for the control of this test. TV-02 will be revised to address the use of the blind flange.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Big Rock Point Nuclear Plant	50-155	97	001	00	2 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

IDENTIFICATION OF EVENT

This event is reportable to the Nuclear Regulatory Commission pursuant to :

10 CFR 50.73(a)(2)(i)(B) - "Any operation prohibited by the plant's Technical Specifications".

REFERENCES

1. Big Rock Point Technical Specification 3.7(d).
3. LER 96-013 Automatic Reactor Scram Initiated by Turbine.
4. TV-02, Containment Integrated Leak Rate Test (ILRT).
5. 10 CFR 50 Appendix J- Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors.

EVENT DESCRIPTION

On January 2, 1997, at 1948 hours, TV-02, was initiated. The purpose of this procedure is to pressurize the containment sphere [NH] through penetration [PEN] H-80 and the associated containment isolation valve VCI-1 [ISV], then measure leakage after 8 hours. When the air compressors were valved in to supply air into the containment sphere, and the test could not be performed as expected, the engineers determined that a blind flange had been installed between system piping flanges. The blind flange is believed to have been in this location since 1992, the date of the last ILRT. This condition is contrary to Big Rock Point Technical Specification 3.7(d) because four Local Leak Rate Tests (LLRTs) conducted on this penetration since 1992 were not performed in accordance with Appendix J criteria. The H-80 containment penetration was tested in this condition and no leakage requirements were exceeded. However piping downstream of VCI-1 was not properly vented, therefore the normally closed VCI-1 was not tested in accordance with Appendix J requirements.

ROOT CAUSE

A root cause analysis was performed and concluded that an inadequate procedure precipitated the event. The procedure does not provide the appropriate amount of detail for the outboard blind flange installation, which can lead to a nonconforming test.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Big Rock Point Nuclear Plant	50-155	97	001	00	3 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

CORRECTIVE ACTION

Immediate

The blind flange was removed, a new gasket was inserted, a successful LLRT was performed on H-80, and containment sphere pressurization resumed.

Long Term

Station procedure TV-02, Containment Integrated Leak Rate Test (ILRT), will be revised to provide direction with respect to the outboard blind flange on containment penetration H-80.

THIS ACTION WILL BE COMPLETED BY MAY 1, 1997

SAFETY SIGNIFICANCE

This event had no safety significance since containment integrity leakage requirements have been maintained. There have been four successful local leak rate tests (since the 1992 ILRT) on penetration H-80. With the blind flange installed, (see Figure 1), VCI-1 is not vented downstream as required by Appendix J. However, the measured leakage from H-80 in this configuration (testing the blind flange instead of VCI-1) did not affect the containment leak rate summary results. Therefore, while the LLRTs were not performed in accordance with approved testing methods with respect to VCI-1, containment penetration leakage requirements have been satisfied (as demonstrated by the four successful LLRT since 1992), remaining below the Appendix J acceptance criteria.

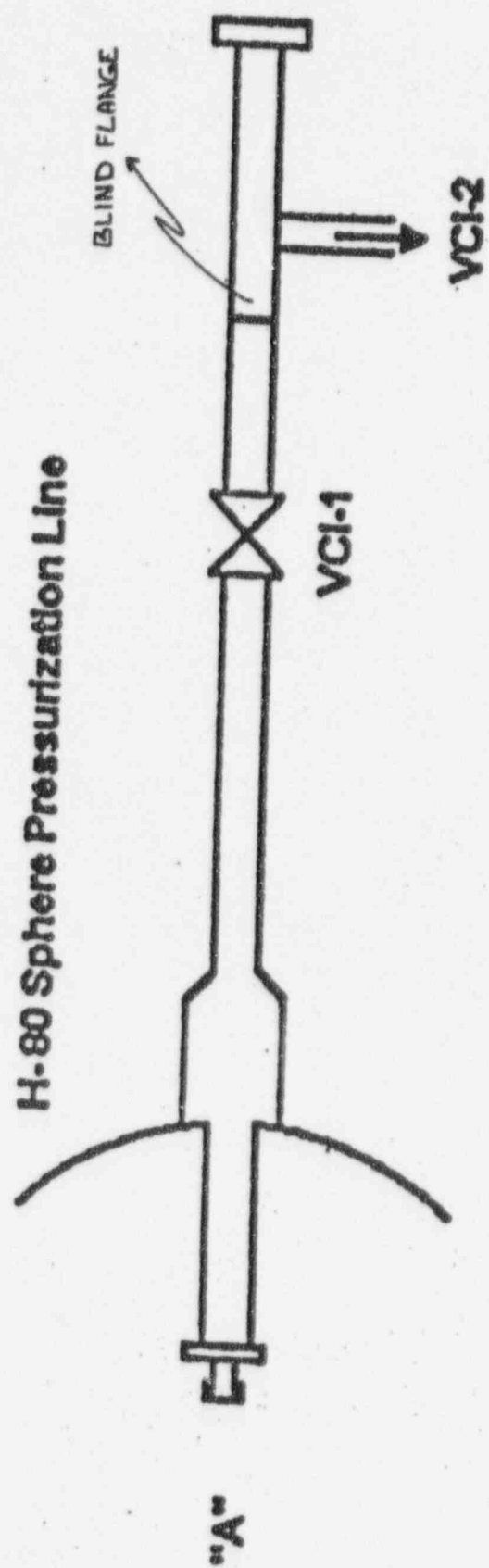


FIGURE 1