

LICENSEE EVENT REPORT (LER)

APPROVED OMS NO. 3180-0104
EXPIRES 6/30/85

FACILITY NAME (1)										DOCKET NUMBER (2)										PAGE 1																													
INDIAN POINT, UNIT 2										0 5 0 0 0 2 4 7 1 0 F Q 3																																							
TITLE (4)																																																	
MODIFIED SERVICE WATER PUMP SEISMIC RESTRAINTS																																																	
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																			
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES										DOCKET NUMBER (8)												
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OPERATING MODE (9)										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)																																							
POWER LEVEL (10) 1 1 0 1 0										20.402(b)										20.406(a)										50.73(a)(2)(iv)										73.71(a)									
										20.406(a)(1)(i)										50.73(a)(1)										50.73(a)(2)(iv)										73.71(a)									
										20.406(a)(1)(ii)										50.73(a)(2)										50.73(a)(2)(iv)										X OTHER (Specify in Abstract below and in Test, NRC Form 306A)									
										20.406(a)(1)(iii)										50.73(a)(2)(i)										50.73(a)(2)(iii)(A)										(For Info Only)									
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										20.406(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(iv)																			
LICENSEE CONTACT FOR THIS LER (12)																																																	
NAME																				TELEPHONE NUMBER																													
JOHN R. ELLWANGER																				AREA CODE 9 1 4 5 2 6 - 5 1 1 8 1 2																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC				CAUSE		SYSTEM		COMPONENT		MANUFACTURER		REPORTABLE TO NRC																													
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SUPPLEMENTAL REPORT EXPECTED (14)																																																	
YES (If yes, complete EXPECTED SUBMISSION DATE)																				NO																													
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ABSTRACT (Limit to 1000 words, i.e., approximately fifteen single-space typewritten lines) (16)

On July 11, 1985, an inspection of the six service water pumps indicated a gap between the seismic restraint and a collar mounted on the vertical pumps in excess of that specified on the original structural drawing. In addition, the restraints did not align vertically with the seismic collars mounted on the pump casing. In response to these findings, on July 11, 1985 four plates (each representing a 90° sector) were welded to the existing plates for each pump, which eliminated the excessive gap and corrected the vertical alignment problem.

A stress analysis is underway to determine whether the service water pumps in their "as found" condition would have remained operable during a design basis seismic event. At the conclusion of this analysis a decision will be made regarding the reportability of this event per the provisions of 10 CFR 50.73. This report is being filed in advance of a determination of reportability in order to comply with the 30 day reporting period requirement of 10 CFR 50.73 should that section be ultimately found to apply.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO. 3190-0104

EXPIRES 8/31/85

FACILITY NAME (1) INDIAN POINT, UNIT 2	DOCKET NUMBER (2) 0 5 0 0 0 2 4 7	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (if more space is required, use additional NRC Form 368A) (17)

Plant and System Identification:

Westinghouse 4-loop Pressurized Water Reactor - 900 MWE

Identification of Occurrence:

Modified seismic restraints on Service Water Pumps

Event Date: 7/11/85Reportability Determination: Not applicable at this timeReport Due Date: 8/10/85

This report was initiated by SOR 85-293

Description of Occurrence

On July 11, 1985, while at full power operation, an inspection was made of the seismic restraints on the six (6) Service Water Pumps. The inspection was initiated at the request of the NRC resident inspector who was aware of prior modifications to similar restraints at I.P.-3. The original drawings for pump installation and alignment with the seismic restraint permitted no more than 1/16 of inch gap between the seismic restraint and a collar mounted on the vertical pump. The collar and restraint were to align vertically without an offset. The "as found" conditions consisted of a gap of approximately two inches and a coincident misalignment in the vertical direction. The pumps were declared inoperable and an LCO entered. Prior to completion of the LCO, emergency repairs were completed and the plant was returned to full power operation.

A dynamic structural analysis will form the basis to determine whether the Service Water Pumps in their "as found" condition would have remained operable in the event of a Design Basis Earthquake. It is anticipated that the results of this analysis will be available by mid-November 1985, at which time the reportability of the event will be determined and an updated LER may be filed.

Apparent Cause of Occurrence

An evaluation of maintenance records was performed to determine a possible cause. A November 20, 1975 maintenance request addressed

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMS NO. 3190-0104

EXPIRES 8/21/85

FACILITY NAME (1)

INDIAN POINT, UNIT 2

DOCKET NUMBER (2)

0 5 0 0 0 2 4 7

LER NUMBER (6)

YEAR

SEQUENTIAL
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PAGE (3)

0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 368a) (57)

excessive vibration on #26 Service Water Pump. An investigation at that time indicated pump misalignment with the casing in hard contact (i.e., pre-loading) with the horizontal restraint. The restraint was relaxed and vibration decreased to an acceptable level.

It is believed that in the process of "relaxing" the restraint, the gap was enlarged by the cutting of metal from the restraint plate. This same process was evidently employed on all six pumps. The vertical misalignment observed was evidently due to shimming of the pumps during replacement or maintenance.

Analysis of Occurrence

The dynamic stress analysis is estimated to be completed by mid-November 1985. If the analysis determines the event to be reportable, an updated LER will be submitted.

Corrective Action

If the event is ultimately determined to be reportable, then appropriate corrective action has already been taken. If the event is determined not to be reportable, then the action taken to date is deemed to be additional conservatism with respect to adequacy of the service water pump restraints.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

August 9, 1985

Re: Indian Point Unit No. 2
Docket No. 50-247
LER-85-007-00

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sirs:

The attached Licensee Event Report LER-85-007-00 is hereby submitted for information only, in advance of a determination of reportability in order to comply with the requirements of 10 CFR Part 50.73 should that section be ultimately found to apply.

Very truly yours,

John D. O'Toole

attach.

cc: Dr. Thomas Murley,
Regional Administrator - Region I
U. S. Nuclear Regulatory Commission
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