

October 10, 1996

Mr. George A. Hunger, Jr.
Manager-Licensing, MC 62A-1
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box 195
Wayne, PA 19087-0195

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI), LIMERICK GENERATING
STATION, UNITS 1 AND 2 (TAC NOS. M96117 AND M96118)

Dear Mr. Hunger:

On June 26, 1996, PECO Energy Company submitted for review and approval a proposed Technical Specifications change that would incorporate performance based testing in accordance with 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water Cooled Power Reactors," Option B. The NRC staff has reviewed your submittal and has determined that additional information, as stated in the enclosure, is needed by the staff to complete our review.

We request that the licensee provide its response within 30 days, to support our review schedule. If you have any questions on the enclosed RAI, please contact me at (301) 415-1447.

Sincerely,

/S/

Frank Rinaldi, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-352/353

Enclosure: RAI

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, reading "Frank Rinaldi", is written over a horizontal line.

Frank Rinaldi, Project Manager
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-352/353

Enclosure: RAI

cc w/encl: See next page

Mr. George A. Hunger, Jr.
PECO Energy Company

Limerick Generating Station,
Units 1 & 2

cc:

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Board of Supervisors
of Limerick Township
646 West Ridge Pike
Linfield, PA 19468

REQUEST FOR ADDITIONAL INFORMATION

APPENDIX J, OPTION B

LIMERICK GENERATING STATION, UNITS 1 AND 2

1. Provide a discussion of the potential increase in risk due to extending the drywell bypass leakage test interval to 10 years. Address such issues as: (1) increase in risk of overpressurizing containment due to bypass leakage following a severe accident, (2) increase in source terms for various events due to bypass of the suppression pool followed by containment failure, and (3) the possibility of bypass leakage of large amounts of hydrogen to the suppression pool.
2. Are there areas which could affect the drywell bypass leakage which will be inaccessible and therefore not readily inspected visually or not inspected at all?
3. What controls are there over modifications to the drywell/suppression pool interface that could affect leakage?
4. Is there a backup to containment spray? If there are procedures governing the use of spray backup, describe them.
5. List all lines or penetrations between the drywell and the suppression pool which are not subject to Appendix J leak testing requirements. What assurance is there that these are not potential leak paths?
6. The Final Safety Analysis Report demonstrates that the suppression chamber sprays will maintain the pressure below the containment design pressure for the design basis bypass leakage. How much larger could the leakage be before containment spray becomes ineffective?
7. Describe the frequency and type of any non-destructive testing of the liner plate over the diaphragm slab at penetrations and at the circumference where the diaphragm slab intersects the containment wall.

ENCLOSURE