

ATTACHMENT B-1

PROPOSED CHANGES TO APPENDIX A, TECHNICAL SPECIFICATIONS, OF FACILITY OPERATING LICENSE NPF-37 and NPF-66

BYRON NUCLEAR POWER STATION

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TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
<u>6. Main Steam Isolation (Continued)</u>			
85	MS101B*	Main Steam	6
86	MS101C*	Main Steam	6
<u>7. Feedwater Isolation</u>			
76	FW009D*	Main Feedwater	5
76	FW043D**	Main Feedwater	6
79	FW009A*	Main Feedwater	5
79	FW043A**	Main Feedwater	6
84	FW009B*	Main Feedwater	5
84	FW043B**	Main Feedwater	6
87	FW009C*	Main Feedwater	5
87	FW043C**	Main Feedwater	6
99 ##(6)***	FW035D*	Main Feedwater	6
99 ##(7)***	FW039D*	Main Feedwater	6
100 ##(7)***	FW035A*	Main Feedwater	6
100 ##(7)***	FW039A*	Main Feedwater	6
101 ##(4)***	FW035B*	Main Feedwater	6
101 ##(4)***	FW039B*	Main Feedwater	6
102 ##(7)***	FW035C*	Main Feedwater	6
102 ##(7)***	FW039C*	Main Feedwater	6
<u>8. Remote Manual</u>			
68	RH8701A*,#	RH Suction	N.A.
68	RH8701B*,#	RH Suction	N.A.
75	RH8702A*,#	RH Suction	N.A.
75	RH8702B*,#	RH Suction	N.A.
59	SI8881*	Hot Leg Safety Injection	N.A.
73	SI8824*	Hot Leg Safety Injection	N.A.
66	SI8825*	Hot Leg RH Injection	N.A.
60	SI8823*	Cold Leg Safety Injection	N.A.
50	SI8890A*	Cold Leg RH Injection	N.A.
51	SI8890B*	Cold Leg RH Injection	N.A.
26	SI8843*	Cold Leg Safety Injection	N.A.
33	CV8355A*	RCP Seal Injection	N.A.
33	CV8355D*	RCP Seal Injection	N.A.
53	CV8355B*	RCP Seal Injection	N.A.
53	CV8355C*	RCP Seal Injection	N.A.

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
8. <u>Remote Manual</u> (Continued)			
59	SI8802A*	Hot Leg Safety Injection	N.A.
73	SI8802B*	Hot Leg Safety Injection	N.A.
60	SI8835*	Hot Leg Safety Injection	N.A.
50	SI8809A*	RH Cold Leg Injection	N.A.
51	SI8809B*	RH Cold Leg Injection	N.A.
66	SI8840*	Hot Leg Safety Injection	N.A.
100	AF013A*	Feedwater	N.A.
100	AF013E*	Feedwater	N.A.
101	AF013B*	Feedwater	N.A.
101	AF013F*	Feedwater	N.A.
102	AF013C*	Feedwater	N.A.
102	AF013G*	Feedwater	N.A.
99	AF013D*	Feedwater	N.A.
99	AF013H*	Feedwater	N.A.
9. <u>Manual</u>			
37	CV8346*	RCS Loop Fill	N.A.
I3	VQ016	Instrument Penetration	N.A.
I3	VQ017	Instrument Penetration	N.A.
I3	VQ018	Instrument Penetration	N.A.
I3	VQ019	Instrument Penetration	N.A.
15	RY075	Instrument Penetration	N.A.
30	WM190	Make-Up Demin	N.A.
57	FC009	Spent Fuel Pool Cleaning	N.A.
57	FC010	Spent Fuel Pool Cleaning	N.A.
32	FC011	Spent Fuel Pool Cleaning	N.A.
32	FC012	Spent Fuel Pool Cleaning	N.A.
77	MS021D*,#	Main Steam	N.A.
78	MS021A*,#	Main Steam	N.A.
85	MS021B*,#	Main Steam	N.A.
86	MS021C*,#	Main Steam	N.A.
AL	PR002E#	Process Radiation	N.A.
AL	PR033A#	Process Radiation	N.A.
AL	PR033B#	Process Radiation	N.A.
AL	PR002r#	Process Radiation	N.A.
AL	PR033C#	Process Radiation	N.A.
AL	PR033D#	Process Radiation	N.A.

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
9. <u>Manual</u> (Continued)			
99	FW015D*,#	Feedwater # (Steam Generator Recirculation)***	N.A.
100	FW015A*,#	Feedwater # (Steam Generator Recirculation)***	N.A.
101	FW015B*,#	Feedwater # (Steam Generator Recirculation)***	N.A.
102	FW015C*,#	Feedwater # (Steam Generator Recirculation)***	N.A.
10. <u>Check</u>			
28	CV8113	RCP Seal Water Return	N.A.
37	CV8348*	RCS Loop Fill	N.A.
6	W0007A	Chilled Water	N.A.
10	W0007B	Chilled Water	N.A.
21	CC9534	RCP Mtr Brng Return	N.A.
24	CC9518	RCP Thermal Barrier Return	N.A.
25	CC9486	RCP Cooling Wtr Supply	N.A.
1	CS008A	Containment Spray	N.A.
16	CS008B	Containment Spray	N.A.
39	IA091	Instrument Air	N.A.
30	WM191	Make-Up Demin	N.A.
52	PR032	Process Radiation	N.A.
AL	PR002G	Process Radiation	N.A.
AL	PR002H	Process Radiation	N.A.
12	PS231A	Hydrogen Monitor	N.A.
31	PS231B	Hydrogen Monitor	N.A.
27	RY8047	PRT Nitrogen	N.A.
44	RY8046	PRT Make-Up	N.A.
26	SI8815*	Safety Injection	N.A.
50	SI8818A*	Safety Injection	N.A.
50	SI8818D*	Safety Injection	N.A.
51	SI8818B*	Safety Injection	N.A.
51	SI8818C*	Safety Injection	N.A.
59	SI8905A*	Safety Injection	N.A.
59	SI8905D*	Safety Injection	N.A.
60	SI8819A*	Safety Injection	N.A.
60	SI8819B*	Safety Injection	N.A.

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
10. <u>Check</u> (Continued)			
60	SI8819C*	Safety Injection	N.A.
60	SI8819D*	Safety Injection	N.A.
66	SI8841A*	Safety Injection	N.A.
66	SI8841B*	Safety Injection	N.A.
73	SI8905B*	Safety Injection	N.A.
73	SI8905C*	Safety Injection	N.A.
55	SI8968	Safety Injection	N.A.
3*	FP345*	Fire Protection	N.A.
33	CV8368A*	RCP Seal Injection	N.A.
33	CV8368D*	RCP Seal Injection	N.A.
53	CV8368B*	RCP Seal Injection	N.A.
53	CV8368C*	RCP Seal Injection	N.A.
11. <u>S/G Safeties/PORVs</u>			
77	MS013D*	Main Steam	N.A.
77	MS014D*	Main Steam	N.A.
77	MS015D*	Main Steam	N.A.
77	MS016D*	Main Steam	N.A.
77	MS017D*	Main Steam	N.A.
78	MS013A*	Main Steam	N.A.
78	MS014A*	Main Steam	N.A.
78	MS015A*	Main Steam	N.A.
78	MS016A*	Main Steam	N.A.
78	MS017A*	Main Steam	N.A.
85	MS013B*	Main Steam	N.A.
85	MS014B*	Main Steam	N.A.
85	MS015B*	Main Steam	N.A.
85	MS016B*	Main Steam	N.A.
85	MS017B*	Main Steam	N.A.
86	MS013C*	Main Steam	N.A.
86	MS014C*	Main Steam	N.A.
86	MS015C*	Main Steam	N.A.
86	MS016C*	Main Steam	N.A.
86	MS017C*	Main Steam	N.A.
77	MS018D*	Main Steam	20
78	MS018A*	Main Steam	20
85	MS018B*	Main Steam	20
86	MS018C*	Main Steam	20

*Not subject to Type C leakage tests.

**Proper valve operation will be demonstrated by verifying that the valve strokes to its required position.

#May be opened on an intermittent basis under administrative control.

ATTACHMENT B-2

PROPOSED CHANGES TO APPENDIX A, TECHNICAL SPECIFICATIONS, OF FACILITY OPERATING LICENSE NPF-72 and NPF-77

BRAIDWOOD NUCLEAR POWER STATION

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TABLE 3.6-1 (Continued)

CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
<u>6. Main Steam Isolation (Continued)</u>			
85	MS101B*	Main Steam	6
86	MS101C*	Main Steam	6
<u>7. Feedwater Isolation</u>			
76	FW009D*	Main Feedwater	5
76	FW043D*##	Main Feedwater	6
79	FW009A*	Main Feedwater	5
79	FW043A*##	Main Feedwater	6
84	FW009B*	Main Feedwater	5
84	FW043B*##	Main Feedwater	6
87	FW009C*	Main Feedwater	5
87	FW043C*##	Main Feedwater	6
99 ## (76)***	FW035D*	Main Feedwater	6
99 ## (76)***	FW039D*	Main Feedwater	6
100 ## (79)***	FW035A*	Main Feedwater	6
100 ## (79)***	FW039A*	Main Feedwater	6
101 ## (64)***	FW035B*	Main Feedwater	6
101 ## (64)***	FW039B*	Main Feedwater	6
102 ## (67)***	FW035C*	Main Feedwater	6
102 ## (67)***	FW039C*	Main Feedwater	6
<u>8. Remote Manual</u>			
68	RH8701A*,#	RH Suction	N.A.
68	RH8701B*,#	RH Suction	N.A.
75	RH8702A*,#	RH Suction	N.A.
75	RH8702B*,#	RH Suction	N.A.
59	SI8881*	Hot Leg Safety Injection	N.A.
73	SI8824*	Hot Leg Safety Injection	N.A.
66	SI8825*	Hot Leg RH Injection	N.A.
60	SI8823*	Cold Leg Safety Injection	N.A.
50	SI8890A*	Cold Leg RH Injection	N.A.
51	SI8890B*	Cold Leg RH Injection	N.A.
26	SI8843*	Cold Leg Safety Injection	N.A.
33	CV8355A*	RCP Seal Injection	N.A.
33	CV8355D*	RCP Seal Injection	N.A.
53	CV8355B*	RCP Seal Injection	N.A.
53	CV8355C*	RCP Seal Injection	N.A.

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
<u>8. Remote Manual (Continued)</u>			
59	SI8802A*	Hot Leg Safety Injection	N.A.
73	SI8802B*	Hot Leg Safety Injection	N.A.
60	SI8835*	Hot Leg Safety Injection	N.A.
50	SI8809A*	RH Cold Leg Injection	N.A.
51	SI8809B*	RH Cold Leg Injection	N.A.
66	SI8840*	Hot Leg Safety Injection	N.A.
100 #* (7) ***	AF013A*	Feedwater	N.A.
100 #* (7) ***	AF013E*	Feedwater	N.A.
101 #* (7) ***	AF013B*	Feedwater	N.A.
101 #* (7) ***	AF013F*	Feedwater	N.A.
102 #* (7) ***	AF013C*	Feedwater	N.A.
102 #* (7) ***	AF013G*	Feedwater	N.A.
99 #* (7) ***	AF013D*	Feedwater	N.A.
99 #* (7) ***	AF013H*	Feedwater	N.A.
<u>9. Manual</u>			
37	CV8346*	RCS Loop Fill	N.A.
13	VQ016	Instrument Penetration	N.A.
13	VQ017	Instrument Penetration	N.A.
13	VQ018	Instrument Penetration	N.A.
13	VQ019	Instrument Penetration	N.A.
15	RY075	Instrument Penetration	N.A.
30	WM190	Make-Up Demin	N.A.
57	FC009	Spent Fuel Pool Cleaning	N.A.
57	FC010	Spent Fuel Pool Cleaning	N.A.
32	FC011	Spent Fuel Pool Cleaning	N.A.
32	FC012	Spent Fuel Pool Cleaning	N.A.
77	MS021D*,#	Main Steam	N.A.
78	MS021A*,#	Main Steam	N.A.
85	MS021B*,#	Main Steam	N.A.
86	MS021C*,#	Main Steam	N.A.
AL	PP002E#	Process Radiation	N.A.
AL	PH033A#	Process Radiation	N.A.
AL	PR0333#	Process Radiation	N.A.
AL	PR002F#	Process Radiation	N.A.
AL	PR033C#	Process Radiation	N.A.
AL	PR033D#	Process Radiation	N.A.

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
9. <u>Manual</u> (Continued)			
99	FW015D*,#	Feedwater	N.A.
100	FW015A*,#	Feedwater	N.A.
101	FW015B*,#	Feedwater	N.A.
102	FW015C*,#	Feedwater	N.A.
10. <u>Check</u>			
28	CV8113	RCP Seal Water Return	N.A.
37	CV8348*	RCS Loop Fill	N.A.
6	W0007A	Chilled Water	N.A.
10	W0007B	Chilled Water	N.A.
21	CC9534	RCP Mtr Brng Return	N.A.
24	CC9518	RCP Thermal Barrier Return	N.A.
25	CC9486	RCP Cooling Wtr Supply	N.A.
1	CS008A	Containment Spray	N.A.
16	CS008B	Containment Spray	N.A.
39	IA091	Instrument Air	N.A.
30	WM191	Make-Up Demin	N.A.
52	PR032	Process Radiation	N.A.
AL	PR002G	Process Radiation	N.A.
AL	PR002H	Process Radiation	N.A.
12	PS231A	Hydrogen Monitor	N.A.
31	PS231B	Hydrogen Monitor	N.A.
27	RY8047	PRT Nitrogen	N.A.
44	RY8046	PRT Make-Up	N.A.
26	SI8815*	Safety Injection	N.A.
50	SI8818A*	Safety Injection	N.A.
50	SI8818D*	Safety Injection	N.A.
51	SI8818B*	Safety Injection	N.A.
51	SI8818C*	Safety Injection	N.A.
59	SI8905A*	Safety Injection	N.A.
59	SI8905D*	Safety Injection	N.A.
60	SI8819A*	Safety Injection	N.A.
60	SI8819B*	Safety Injection	N.A.

TABLE 3.6-1 (Continued)
CONTAINMENT ISOLATION VALVES

<u>PENETRATION</u>	<u>VALVE NO.</u>	<u>FUNCTION</u>	<u>MAXIMUM ISOLATION TIME (SEC)</u>
10. <u>Check</u> (Continued)			
60	SI8819C*	Safety Injection	N.A.
60	SI8819D*	Safety Injection	N.A.
66	SI8841A*	Safety Injection	N.A.
66	SI8841B*	Safety Injection	N.A.
73	SI8905B*	Safety Injection	N.A.
73	SI8905C*	Safety Injection	N.A.
55	SI8968	Safety Injection	N.A.
34	FP345*	Fire Protection	N.A.
33	CV8368A*	RCP Seal Injection	N.A.
33	CV8368D*	RCP Seal Injection	N.A.
53	CV8368B*	RCP Seal Injection	N.A.
53	CV8368C*	RCP Seal Injection	N.A.
11. <u>S/G Safeties/PORVs</u>			
77	MS013D*	Main Steam	N.A.
77	MS014D*	Main Steam	N.A.
77	MS015D*	Main Steam	N.A.
77	MS016D*	Main Steam	N.A.
77	MS017D*	Main Steam	N.A.
78	MS013A*	Main Steam	N.A.
78	MS014A*	Main Steam	N.A.
78	MS015A*	Main Steam	N.A.
78	MS016A*	Main Steam	N.A.
78	MS017A*	Main Steam	N.A.
85	MS013B*	Main Steam	N.A.
85	MS014B*	Main Steam	N.A.
85	MS015B*	Main Steam	N.A.
85	MS016B*	Main Steam	N.A.
85	MS017B*	Main Steam	N.A.
86	MS013C*	Main Steam	N.A.
86	MS014C*	Main Steam	N.A.
86	MS015C*	Main Steam	N.A.
86	MS016C*	Main Steam	N.A.
86	MS017C*	Main Steam	N.A.
77	MS018D*	Main Steam	20
78	MS018A*	Main Steam	20
85	MS018B*	Main Steam	20
86	MS018C*	Main Steam	20

- *Not subject to Type C leakage tests. *Applicable for Unit 1 after Cycle 7.*
- **Proper valve operation will be demonstrated by verifying that the valve strokes to its required position.
- #May be opened on an intermittent basis under administrative control.
- * * * Not applicable to Unit 2.*

ATTACHMENT C

EVALUATION OF SIGNIFICANT HAZARDS CONSIDERATIONS FOR PROPOSED CHANGES TO APPENDIX A, TECHNICAL SPECIFICATIONS, OF FACILITY OPERATING LICENSES NPF-37, NPF-66, NPF-72 AND NPF-77

Commonwealth Edison Company (ComEd) has evaluated this proposed amendment and has determined that it involves no significant hazards considerations. According to Title 10, Code of Federal Regulations, Section 50, Subsection 92, Paragraph c (10 CFR 50.92 (c)), a proposed amendment to an operating license involves no significant hazards considerations if operation of the facility in accordance with the proposed amendment would not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated; or
2. Create the possibility of a new or different kind of accident from any accident previously evaluated, or
3. Involve a significant reduction in margin of safety.

A. INTRODUCTION

As a result of the planned replacement of the Westinghouse D4 steam generators at Byron Unit 1 and Braidwood Unit 1, changes will be made to the containment isolation piping arrangements at the penetrations associated with the Feedwater (FW) and Auxiliary Feedwater (AF) systems. As a result of these changes, there will be no split FW flow with the replacement steam generators. AF flow will be fed into the main FW piping outside of containment and the existing FW tempering penetration will be used for a new steam generator recirculation system to be used during periods of extended shutdown. Additionally, since the replacement steam generators use a feedring design rather than a preheater design, the FW Isolation Bypass line and associated containment isolation valves will no longer be required. Table 3.6-1 of the Technical Specifications (TS) must be updated to reflect these changes. These changes do not affect the containment isolation capability originally designed to the criteria in 10 CFR 50, Appendix A, General Design Criteria (GDC) 54 through 57 as reflected in the Byron/Braidwood Updated Final Safety Analysis Report (UFSAR).

B. NO SIGNIFICANT HAZARDS ANALYSIS

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Technical Specification 3/4.6.3 establishes the operability requirements for containment isolation valves as required by the Byron and Braidwood Operating Licenses in compliance with General Design Criteria 54 through 57 of Appendix A to 10 CFR 50. The operability of the containment isolation valves ensure that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere. Table 3.6-1 identifies these isolation valves and captures relevant information to ensure these valves remain operable under required conditions.

These proposed changes result in the elimination of the FW Isolation Bypass isolation valves. These isolation valves are not required with the replacement steam generator design. The remaining isolation valves have not been altered in any way, only the piping associated with them has been altered to the revised configuration. These changes do not result in alteration of any containment penetrations.

Failure of the piping between the isolation valve and the containment penetration is considered as an accident initiator. However, all piping changes between the isolation valve and the containment penetrations meet the requirements of the original design.

Therefore, since all original piping design criteria are met and the actual number of containment isolation valves is reduced, the proposed change does not involve a significant increase in the probability of an accident previously evaluated.

Each penetration identified in the proposed change is associated with a closed system inside containment and, as such, is provided containment isolation in accordance with the applicable requirements of GDC 54 through 57. There are four analyzed transients which take credit for feedwater isolation and are, therefore, relevant to this proposed change. These accidents are: 1) feedwater system malfunctions that result in an increase in FW flow, 2) inadvertent opening of a steam generator relief or safety valve, 3) steam system piping failure, and 4) FW system pipe break. All operability requirements for the affected containment isolation valves are unaffected by this proposed change.

The containment isolation valves' functions, system operating conditions, and accident responses are unchanged as a result of the new configuration. Therefore, since all original design criteria are met and each remaining isolation valve continues to provide the same degree of containment isolation as the original design, the proposed change does not involve a significant increase in the consequences of an accident previously evaluated.

2. **The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.**

All modifications associated with the proposed changes will be outside of containment and can be characterized as the rearrangement of piping systems. All piping changes will comply with the original design of the plant and will retain required containment isolation capabilities per the requirements of GDC 54 through 57 as required by the current design basis. Piping configurations within the area of the containment penetration and the containment isolation valves are required to minimize branch connections per guidance in the Standard Review Plan (SRP) Section 3.6.2.

Therefore, since there are no unique configurations or reductions in design requirements, this proposed change does not create the possibility of any new or different kinds of accidents from those previously evaluated.

3. **The proposed change does not involve a significant reduction in a margin of safety.**

The proposed changes to the containment isolation arrangement are being made consistent with the same codes, standards, and isolation criteria as are currently in use at Byron and Braidwood. The containment isolation valves remaining in place following the steam generator replacement are unchanged with regard to their function, capability, reliability, or physical requirements. Containment isolation capability in accordance with GDC 54 through 57 is maintained at current levels of protection for the health and safety of the general public. Therefore, this proposed change does not involve a significant reduction in the margin of safety.

Based on the above, Commonwealth Edison has concluded that these changes involve no significant hazards considerations.

ATTACHMENT D

ENVIRONMENTAL ASSESSMENT FOR PROPOSED CHANGES TO APPENDIX A TECHNICAL SPECIFICATIONS OF FACILITY OPERATING LICENSES NPF-37, NPF-66, NPF-72, AND NPF-77

Commonwealth Edison Company (ComEd) has evaluated this proposed license amendment request against the criteria for identification of licensing and regulatory actions requiring environmental assessment in accordance with Title 10, Code of Federal Regulations, Part 51, Section 21 (10 CFR 51.21). ComEd has determined that this proposed license amendment request meets the criteria for a categorical exclusion set forth in 10 CFR 51.22(c)(9). This determination is based upon the following:

1. The proposed licensing action involves the issuance of an amendment to a license for a reactor pursuant to 10 CFR 50 which changes a requirement with respect to installation or use of a facility component located within the restricted area, as defined in 10 CFR 20, or which changes an inspection or a surveillance requirement. This proposed license amendment request will allow ComEd to delete four feedwater bypass valves, change the containment penetrations associated with eight feedwater valves, change the containment penetrations associated with eight auxiliary feedwater valves, and change the function of four feedwater valves to a steam generator recirculation function. All of these changes affect only Unit 1 at Byron and Braidwood;
2. this proposed license amendment request involves no significant hazards considerations;
3. there is no significant change in the types or significant increase in the amounts of any effluent that may be released offsite; and
4. there is no significant increase in individual or cumulative occupational radiation exposure.

Therefore, pursuant to 10 CFR 51.22(b), neither an environmental impact statement nor an environmental assessment is necessary for this proposed license amendment request.

ATTACHMENT E

FIGURE 1: EXISTING ISOLATION ARRANGEMENT

FIGURE 2: REVISED ISOLATION ARRANGEMENT
(UNITS 1 ONLY)

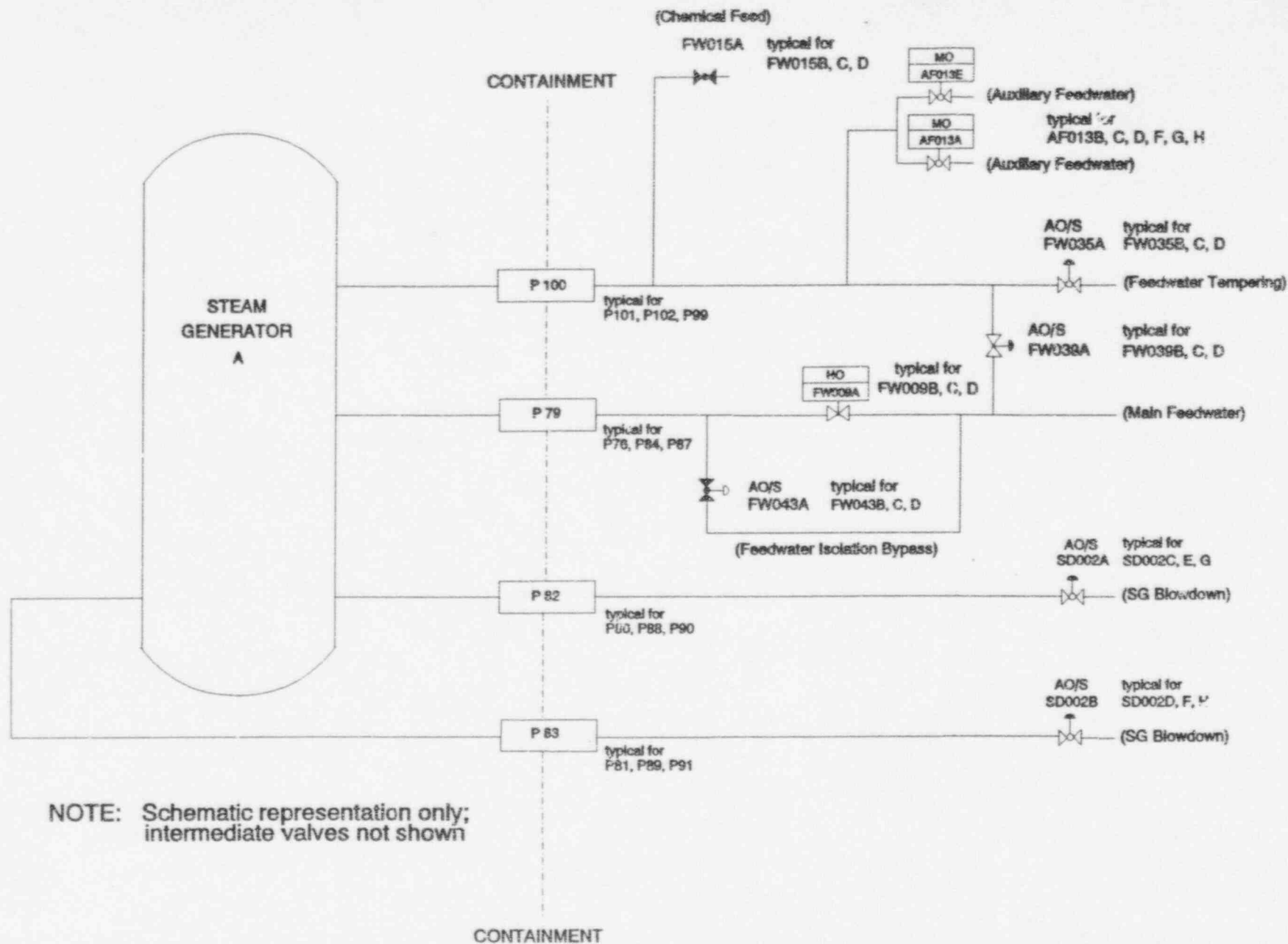


FIGURE 1
EXISTING ISOLATION ARRANGEMENT

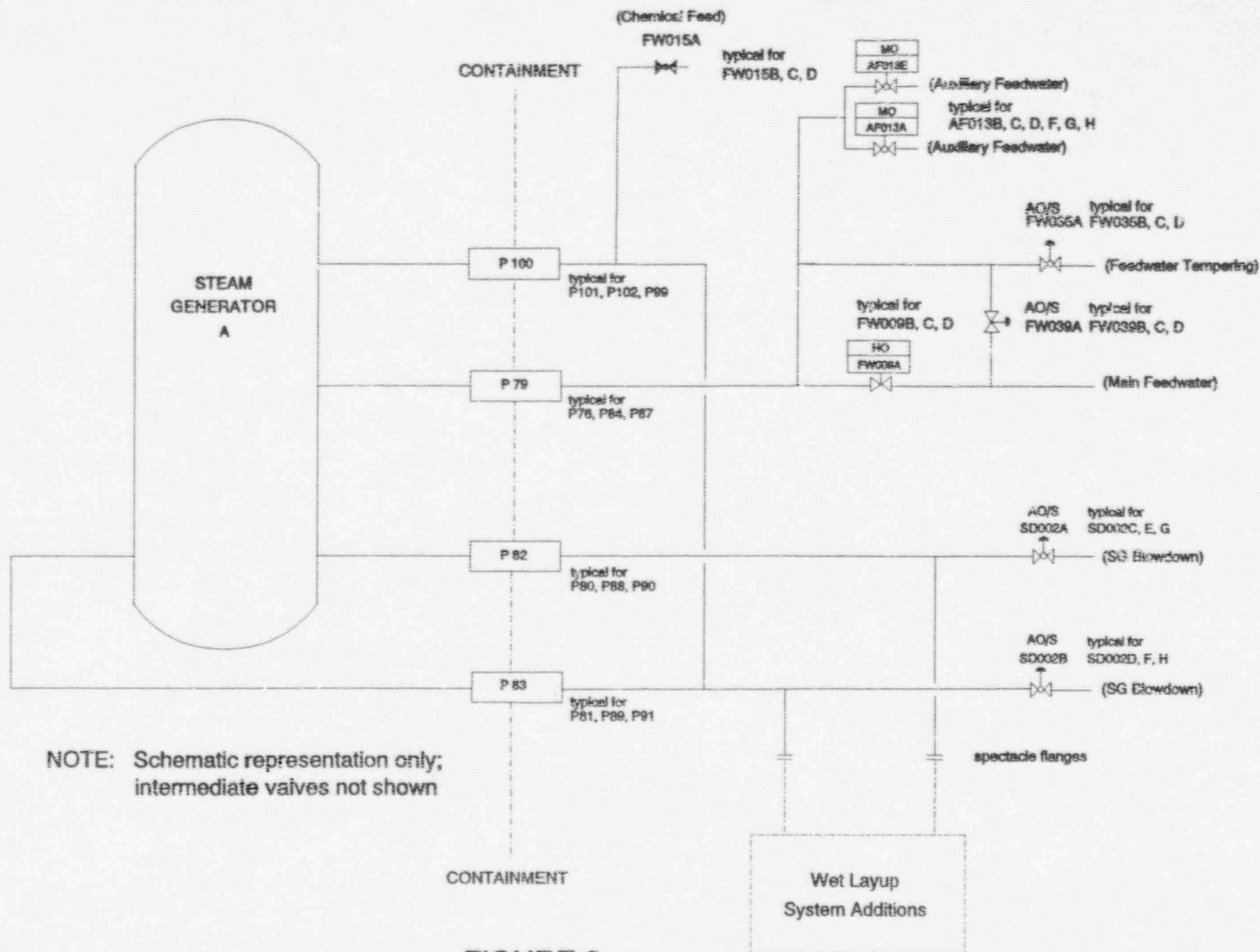


FIGURE 2
REVISED ISOLATION ARRANGEMENT
(UNIT'S 1 ONLY)