



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 30, 1995

Mr. Y. Y. Yung, Chairman
VIPRE-01 Maintenance Group
Washington Public Power Supply System
P.O. Box 968
Richland, WA 99352

Dear Mr. Yung:

SUBJECT: ACCEPTANCE FOR REFERENCING OF THE MODIFIED LICENSING TOPICAL
REPORT, EPRI NP-2511-CCM, REVISION 3, "VIPRE-01: A THERMAL-
HYDRAULIC ANALYSIS CODE FOR REACTOR CORES", (TAC NO. M79498)

We have completed our review of the subject topical report submitted by the VIPRE-01 Maintenance Group (VMG) by letter dated February 28, 1990. We find the report to be acceptable for referencing in license applications to the extent specified and under the limitations delineated in the report and the associated NRC evaluation, which is enclosed. The evaluation defines the basis for acceptance of the report.

We do not intend to repeat our review of the matters described in the report and found acceptable when the report appears as a reference in the license applications, except to assure that the material presented is applicable to the specific plant involved. Our acceptance applies only to the matters described in the report.

In accordance with procedures established in NUREG-390, it is requested that VMG publish accepted versions of this report, proprietary and non-proprietary, within three months of receipt of this letter. The accepted versions shall incorporate this letter and the enclosed evaluation between the title page and the abstract. The accepted versions shall include an -A (designating accepted) following the report identification symbol.

Should our criteria or regulations change such that our conclusions as to the acceptability of the report are invalidated, VMG and/or the applicants referencing the topical report will be expected to revise and resubmit their respective documentation, or submit justification for continued effective applicability of the topical report without revision of their respective documentation.

Sincerely,

Ashok C. Thadani, Assistant Director
Division of System Safety and Analysis
Office of Nuclear Reactor Regulation

Enclosure:
NRC Evaluation

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CF SUBJ 9697 CF
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Enclosure 2