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C. K. McCoy
Vice President, Nuclear
Vogtle Project



February 18, 1993

ELV-05235

Docket No. 50-425

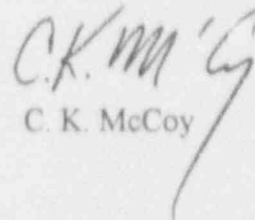
U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Gentlemen:

VOGTLE ELECTRIC GENERATING PLANT
LICENSEE EVENT REPORT
COLD OVERPRESSURIZATION PROTECTION SETPOINTS
FOUND TO BE OUTSIDE OF TECHNICAL SPECIFICATION LIMITS

In accordance with the requirements of 10 CFR 50.73, Georgia Power Company submits the enclosed report related to an event which was discovered on January 29, 1993.

Sincerely,



C. K. McCoy

CKM/NJS

Enclosure: LER 50-425/1993-001

xc: Georgia Power Company
Mr. W. B. Shipman
Mr. M. Sheibani
NORMS

U. S. Nuclear Regulatory Commission
Mr. S. D. Ebnetter, Regional Administrator
Mr. D. S. Hood, Licensing Project Manager, NRR
Mr. B. R. Bonser, Senior Resident Inspector, Vogtle

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) VOGTLE ELECTRIC GENERATING PLANT - UNIT 2 DOCKET NUMBER (2) 05000425 PAGE (3) 1 OF 3

TITLE (4) COLD OVERPRESSURIZATION PROTECTION SETPOINTS FOUND TO BE OUTSIDE OF TECHNICAL SPECIFICATION LIMITS

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQ NUM	REV	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
01	29	93	93	001	00	02	18	93			05000
											05000

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR (11)

OPERATING MODE (9)	1	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL	100	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	Abstract below)
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
MEHDI SHEIBANI, NUCLEAR SAFETY AND COMPLIANCE	AREA CODE 706 826-3209

COMPLETE ONE LINE FOR EACH FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORT TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORT TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (if yes, complete EXPECTED SUBMISSION DATE)	X NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	X				

ABSTRACT (16)

On January 29, 1993, plant personnel found that the low temperature lift settings of the Unit 2 power operated relief valves (PORVs) were incorrect and that these valves had, on several previous occasions, been relied on to meet Technical Specification (TS) requirements for the cold overpressurization protection system (COPS). Therefore, the unit had operated in a condition prohibited by the TS on each of these occasions.

The cause of this problem was inadequate procedures. Setpoint information intended for use in Unit 1 was erroneously used in preparing the Unit 2 calibration procedures. The Unit 2 COPS setpoints have been corrected.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQ NUM	REV			
VOGTLE ELECTRIC GENERATING PLANT - UNIT 2	05000425	93	001	00	2	OF	3

TEXT

A. REQUIREMENT FOR REPORT

This report is required per 10 CFR 50.73 (a)(2)(i) because the unit operated in a condition prohibited by the Technical Specifications (TS) when calibrations and surveillance tests were improperly performed using erroneous setpoint values and were relied on to comply with TS requirements.

B. UNIT STATUS AT TIME OF EVENT

At the time of the discovery of this event, Unit 2 was operating in Mode 1 (power operation) at 100 percent of rated thermal power. Other than that described herein, there was no inoperable equipment which contributed to the occurrence of this event.

C. DESCRIPTION OF EVENT

On January 28, 1993, an engineer was reviewing calibration data for the cold overpressure protection system (COPS) instrumentation as a followup to a discovery at another power plant that COPS setpoints had not taken piping losses into consideration. The engineer found that the setpoints used for determining Unit 2 COPS calibration values were incorrect because setpoints intended for use in Unit 1 were inadvertently used in Unit 2 procedures. The effect was that since initial unit startup, the low temperature lift settings for the power operated relief valves (PORVs) were nonconservative and would have had the PORVs lift at pressures up to 100 psi higher than the intended values. In Modes 4, 5, or 6, TS 3.4.9.3 requires either the PORVs or the residual heat removal (RHR) suction relief valves to be operable, or else a reactor coolant system vent with a specific capacity must be provided. The unit shift supervisor was notified, and since the unit was not in one of the applicable modes, an "information" limiting condition for operation (LCO) was written.

On January 29, 1993, a review of shift logs found that the PORVs had been relied on to meet TS 3.4.9.3 requirements on several previous occasions when the unit had been in Modes 4, 5, or 6. Therefore, the unit had operated in a condition prohibited by the TS on each of these occasions.

D. CAUSE OF EVENT

The direct cause of this problem was inadequate procedures. This was the result of inadequate implementation of PORV low temperature lift setpoint changes into plant procedures during the construction and startup of Unit 2 in 1988-1989. A vendor-supplied setpoint document originally contained one COPS setpoint table intended for use in Unit 1, but not identified as Unit 1 specific. This document was used to determine both Unit 1 and Unit 2 setpoints. Later, but prior to initial Unit 2 startup, the setpoint document was revised to include both Unit 1 and Unit 2 specific COPS setpoints. However, the specific Unit 2 COPS setpoints were not identified during document reviews as a physical change to the plant, and they were not incorporated into Unit 2 procedures prior to the commencement of Unit 2 operation.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQ NUM	REV			
VOGTLE ELECTRIC GENERATING PLANT - UNIT 2	05000425	93	001	00	3	OF	3

TEXT

E. ANALYSIS OF EVENT

A reevaluation by the NSSS vendor determined that the incorrect setpoints fell within the operating criteria of ASME Section III, Appendix G. The original analysis was based on 16 effective full power years (EFPY) of operation using the correct COPS setpoints, but the reevaluation was based on the actual 3.17 EFPY and the incorrect setpoints. Therefore, this condition would not have caused a challenge to the integrity of the reactor vessel had a cold overpressurization event occurred. Furthermore, no actual event occurred which required COPS to actuate. Based on these considerations, there was no adverse effect on plant safety or on the health and safety of the public as a result of this condition.

F. CORRECTIVE ACTIONS

1. The proper scaling calculations have been completed for Unit 2 and the two procedures involved have been changed to reflect the corrected setpoints.
2. The Unit 2 COPS channels have been recalibrated to incorporate the correct setpoints.
3. A review of other setpoints from the plant's Precautions, Limitations and Setpoints (PLS) manual found no similar conditions.

G. ADDITIONAL INFORMATION

1. Failed Components

None

2. Previous Similar Events

None

3. Energy Industry Identification System Code

Reactor Coolant System - AB

Residual Heat Removal System - BP