GULF STATES UTILITIES COMPANY

RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70775

AREA CODE 504 635-6094 345-8651

February 11, 1993

RBG- 38134
File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

River Bend Station - Unit 1
Docket No. 50-458

Please find enclosed Licensee Event Report No. 93-001 for River Bend Station - Unit 1. This report is submitted pursuant to 10CFR50.73.

Sincerely,

W. H. Odell
Manager - Oversight
River Bend Nuclear Group

LAE/JPS/FRC/JAD/kvm

180013

IE22

cc: U.S. Nuclear Regulatory Commission
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011

NRC Resident Inspector
P.O. Box 1051
St. Francisville, LA 70775

INPO Records Center
1100 Circle 75 Parkway
Atlanta, GA 30339-3064

Mr. C.R. Oberg
Public Utility Commission of Texas
7800 Shoal Creek Blvd., Suite 400 North
Austin, TX 78757

Department of Environmental Quality
Radiation Protection Division
P.O. Box 82135
Baton Rouge, LA 70884-2135
ATTN: Administrator

NRC FORM 366 (5-92) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
RIVER BEND STATION	05000 458	1 OF 7

TITLE (4) RWCU ISOLATION SETPOINT ALLOWABLE VALVE WAS EXCEEDED DUE TO FAULTY THERMOCOUPLE TESTER.

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
01	13	93	93	-- 001 --	00	02	12	93		05000
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)			
POWER LEVEL (10)	100	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	(Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
L.A. ENGLAND, DIRECTOR - NUCLEAR LICENSING	(504) 381-4145

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On January 13, 1993 at 0745, with the plant at 100 percent power (Operational Condition 1), the isolation setpoint for temperature unit 1E31-N620A, in the reactor water cleanup system (RWCU), was determined to be above its Technical Specification (TS) allowable value. The trip setpoint value was set very close to the TS allowable value in December 1992. In addition, drifting occurred during the time between the surveillances, resulting in exceeding the TS allowable value. Therefore, the minimum number of operable channels per trip system required by TS Table 3.3.2-1 was not maintained and TS 3.3.2.b was violated. This report is submitted pursuant to 10CFR50.73(a)(2)(i)(b) as operation prohibited by the plant Technical Specifications.

The root cause of this event is failure of a thermocouple tester with ineffective review of STP data. The Technician, Control Operating Foreman, and the Section Supervisor all failed to question the validity of the measurement and test equipment (M&TE) based on the low as-found values.

REQUIRED NUMBER OF DIGITS/CHARACTERS
FOR EACH BLOCK

BLOCK NUMBER	NUMBER OF DIGITS/CHARACTERS	TITLE
1	UP TO 46	FACILITY NAME
2	8 TOTAL 3 IN ADDITION TO 05000	DOCKET NUMBER
3	VARIES	PAGE NUMBER
4	UP TO 76	TITLE
5	6 TOTAL 2 PER BLOCK	EVENT DATE
6	7 TOTAL 2 FOR YEAR 3 FOR SEQUENTIAL NUMBER 2 FOR REVISION NUMBER	LER NUMBER
7	6 TOTAL 2 PER BLOCK	REPORT DATE
8	UP TO 18 -- FACILITY NAME 8 TOTAL -- DOCKET NUMBER 3 IN ADDITION TO 05000	OTHER FACILITIES INVOLVED
9	1	OPERATING MODE
10	3	POWER LEVEL
11	1 CHECK BOX THAT APPLIES	REQUIREMENTS OF 10 CFR
12	UP TO 50 FOR NAME 14 FOR TELEPHONE	LICENSEE CONTACT
13	CAUSE VARIES 2 FOR SYSTEM 4 FOR COMPONENT 4 FOR MANUFACTURER NPRDS VARIES	EACH COMPONENT FAILURE
14	1 CHECK BOX THAT APPLIES	SUPPLEMENTAL REPORT EXPECTED
15	6 TOTAL 2 PER BLOCK	EXPECTED SUBMISSION DATE

REPORTED CONDITION

On January 13, 1993 at 0745, with the plant at 100 percent power (Operational Condition 1), the isolation setpoint for RWCU (*CE*) equipment area high temperature trip unit 1E31-N620A (*TS*), was determined to be above its Technical Specification allowable value. This determination was made during performance of surveillance test procedure (STP)-207-5500. A review of the previous monthly STP performance revealed that an error occurred on December 17, 1992 which resulted in 1E31-N620A (*TS*) being set above the TS trip setpoint and extremely close to its TS allowable value. Further research showed that the test equipment utilized in this STP was faulty, resulting in the error. The trip unit drifted above the allowable value during the time interval between the STP performances in December and January. Since the trip setpoint value was set very close to the TS allowable value in December 1992, GSU concludes that the trip unit drift occurred shortly after the improper setting. Therefore, the minimum number of operable channels per trip system required by TS Table 3.3.2-1 was not maintained and TS 3.3.2.b was violated. This report is submitted pursuant to 10CFR50.73(a)(2)(i)(b) as operation prohibited by the plant Technical Specifications.

INVESTIGATION

STP-207-5500 performs a functional of the RWCU equipment area high temperature trip units for channel 'A'. During the December 1992 performance of this STP, the as-found value for all seven trip units was significantly lower than the trip setpoint value, and the setpoints for all of the trip units were subsequently raised to be consistent with the trip setpoints specified in Table 3.3.2-2 of the Technical Specifications. When STP-207-5500 was performed in January 1993, all seven trip units were determined to be above their trip setpoints with 1E31-N620A (*TS*) being above its Technical Specification allowable value.

Post calibration of TCT-001A (a thermocouple tester utilized for temperature simulation) revealed that the instrument was reading 3.1 degrees F below the actual value that it was being exposed to, thus supplying erroneously low as-found values in December 1992. When these values were observed, the I&C technician increased the trip setpoints commensurate with the values specified in the Technical Specifications. Temperature unit 1E31-N620A was moved from an as-found value 101.0 degrees F to an as-left value of 104.1 degrees F, an increase of 3.1 degrees F. This increase over the as-found value is precisely the test equipment error detected when calibrating TCT-001A in January 1993; therefore, its setpoint was actually 107.2 degrees F. Note that the TS allowable is 107.5 degrees F. When STP-207-5500 was performed in January 1993, 1E31-N620A (*TS*) was found to be 108 degrees F which is .5 degrees F above its Technical Specification allowable value. Thus, the total drift was 0.8 degrees F. The erroneous increase in the trip setpoint accompanied

with a minor amount of instrument drift resulted in the trip unit exceeding its TS allowable value during the time between the December and January tests.

ROOT CAUSE

The root cause of this event is failure of a thermocouple tester with ineffective review of STP data. The Technician, Control Operating Foreman, and the Section Supervisor failed to question the validity of the measurement and test equipment (M&TE) based on the low as-found values.

A review of previous LERs revealed no similar events.

CORRECTIVE ACTION

Thermocouple tester TCT-001A was evaluated to determine its potential effect on other TS items. A review of the STP history revealed that this measurement and test equipment (M&TE) acquired its zero shift error on 12/03/92. The December 1992 performances of STP-207-5255, and STP-207-5254 were all affected by the inaccuracies in this instrument. These surveillances are on the residual heat removal (RHR) (*BO*) equipment area high temperature trip units (*TS*). STP-207-5500 was the only STP in which the Technical Specifications were violated. All three STPs were performed in January 1993, and the improper settings were corrected.

Surveillance test procedure (STP)-207-5500 was satisfactorily completed on January 13, 1993, restoring channel 'A'. The system is now in service with both channels operable. Maintenance personnel responsible for M&TE usage will receive training on this event. In addition, Operations will perform training on this event during licensed operator requalification training with an estimated completion date of 08/01/93.

SAFETY ASSESSMENT

Trip unit 1E31-N620A (*TS*), which is part of the RWCU leak detection system, causes closure of the reactor water cleanup isolation valves when a high temperature is detected in the RWCU heat exchanger room. Since other forms of leak detection (i.e. high system differential flow) are in place, sufficient safety margin existed to ensure proper isolation of the RWCU system if a leak were to develop.

Note: Energy industry identification codes are indicated in the text as (*XX*).