

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
D. C. COOK NUCLEAR PLANT, UNIT 1DOCKET NUMBER (2)  
0 5 0 0 0 3 1 5 1 OF 0 1

PAGE (3)

TITLE (4)  
INCORRECT CALIBRATION OF RESIDUAL HEAT REMOVAL FLOW INSTRUMENTATION

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)	
07	09	85	85	031	00	08	08	85	D.C. COOK - UNIT 2	05000316	
										05000	

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)									
POWER LEVEL (10)	0100	20.402(b)		20.406(a)		50.73(a)(2)(iv)		73.71(b)			
		20.406(a)(1)(i)		50.38(a)(1)		50.73(a)(2)(v)		73.71(c)			
		20.406(a)(1)(ii)		50.38(a)(2)		50.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 388A)			
		20.406(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(vii)(A)					
		20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(vii)(B)					
		20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)

NAME  
A. A. BLIND - ASSISTANT PLANT MANAGER

TELEPHONE NUMBER

AREA CODE  
61646515901

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

X YES (If yes, complete EXPECTED SUBMISSION DATE)

NO

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR  
09 01 85

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ON JULY 9, 1985, AT 1130 HOURS WITH UNIT 1 SHUTDOWN FOR REFUELING WITH NO FUEL IN THE CORE AND UNIT 2 AT 100 PERCENT REACTOR THERMAL POWER, IT WAS DETERMINED THAT UNIT 1 RESIDUAL HEAT REMOVAL (RHR) HOT LEG PROCESS FLOW INSTRUMENTATION (IFI-311 AND 321) WAS INDICATING FLOWS HIGHER THAN ACTUALLY EXISTED. SINCE IFI-311 AND 321 WERE USED FOR ASCERTAINING COMPLIANCE WITH THE TECHNICAL SPECIFICATIONS (T/S) DURING THE NORMAL COOLDOWN CONFIGURATION OF RHR IT WAS RECOGNIZED THAT WHEN THE INSTRUMENT WAS USED FOR THAT PURPOSE, THE ACTUAL RHR FLOW MAY HAVE BEEN BELOW THE T/S MINIMUM REQUIREMENTS.

ON JULY 15, 1985, AT 1600 HOURS (UNIT 1 STATUS UNCHANGED, UNIT 2 AT 95 PERCENT REACTOR THERMAL POWER), A SIMILAR DISCREPANCY WAS DISCOVERED ON RHR COOLDOWN FLOW INDICATION (IFI-335). AS WITH INSTRUMENTS IFI-311 AND 321, THE INDICATED FLOW EXCEEDED THE ACTUAL FLOW. THE ERROR WAS INITIALLY BELIEVED TO HAVE BEEN AN ERROR IN THE SCALE ASSOCIATED WITH THE INSTRUMENTS. FURTHER INVESTIGATION HAD SHOWN THAT IN ADDITION TO THE SCALE ERROR, THERE WAS A PROBLEM WITH THE TEMPERATURE ASSUMED IN THE CALIBRATION AS WELL, SINCE THE INSTRUMENTS WERE USED AT TEMPERATURES SIGNIFICANTLY BELOW THAT WHICH THEY WERE CALIBRATED FOR.

AS A RESULT, PLANT OPERATION MAY NOT HAVE BEEN IN COMPLIANCE WITH T/S 3.1.1.3 AND 3.9.8.1, WHICH REQUIRE A MINIMUM RHR FLOW OF 3000 GPM DURING BORON DILUTION AND REFUELING RESPECTIVELY. HOWEVER, THE RHR FLOW LIMITS CURRENTLY IN PLACE HAVE BEEN CORRECTED TO COMPENSATE FOR THIS DISCREPANCY.

THIS REPORT IS BEING SUBMITTED ON AN INTERIM BASIS PENDING COMPLETION OF THE EVENT INVESTIGATION. NO OTHER ERRONEOUS CALIBRATION DATA HAS BEEN IDENTIFIED AT THIS TIME.

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PDR ADDOCK 05000315  
S PDR