



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
WISCONSIN ELECTRIC POWER COMPANY  
POINT BEACH NUCLEAR PLANT, UNIT NOS. 1 AND 2  
DOCKET NOS. 50-266 AND 50-301  
GENERIC LETTER 83-28, ITEMS 3.1.1, 3.1.2, 4.1 and 4.5.1

1.0 Introduction

A. Background Information

Subsequent to the anticipated transient without scram (ATWS) events at the Salem Nuclear Power Plant, the Commission reviewed intermediate term actions to be taken by the licensees. The actions taken were developed on the basis of information contained in NUREG-1000, "Generic Implications of ATWS Events at the Salem Nuclear Power Plant." On July 8, 1983, Generic Letter (GL) 83-28 was issued by NRR. The letter identified NRC positions developed from review of the Salem ATWS events. These positions are related to reactor trip system reliability and general management capability. The specific GL 83-28 items covered by this Safety Evaluation are Sections 3.1.1 and 3.1.2, Post-Maintenance Testing (Reactor Trip System Components); 4.1, Reactor Trip System Reliability (Vendor-Related Modifications); and 4.5.1, Reactor Trip System Reliability (System Functional Testing).

B. Licensee's Response to Generic Letter (GL) 83-28

By letter dated November 7, 1983, Wisconsin Electric Power Company responded to specific items of GL 83-28. The licensee summarized the results of the requested review and concluded that all items have been appropriately addressed.

C. Scope of Review

The staff's review consisted of an evaluation of the response to determine if the requirements of GL 83-28 have been satisfied. The evaluation for items 3.1.1, 3.1.2, 4.1 and 4.5.1 is given below.

2.0 Evaluation

A. Items 3.1.1 and 3.1.2, Post Maintenance Testing (Reactor Trip System Components)

The review criteria for these items require that the licensee submit a statement indicating that he has reviewed plant test and maintenance procedures and Technical Specifications to ensure that post-maintenance operability testing of safety-related components in the reactor trip system is required. Also, the licensee's statement should contain a verification that vendor recommended test guidance has been reviewed, evaluated, and where appropriate, included in the test and maintenance procedures or the Technical Specifications. The staff has evaluated the licensee's November 7, 1983 submittal for this item and has determined it to be adequate in content.

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B. Item 4.1, Reactor Trip System Reliability (Vendor-Related Modifications)

The review criteria for this item requires that the licensee submit a statement indicating that he has reviewed all vendor recommended reactor trip breaker modifications and determined that (1) each modification has been implemented or (2) a written evaluation which specifies the technical reasons for not implementing the modification exists. The staff has evaluated the licensee's November 7, 1983 submittal for this item and has determined it to be adequate in content.

C. Item 4.5.1, Reactor Trip System Reliability (System Functional Testing)

The review criteria for this item requires that the licensee submit a statement committing to independent, on-line functional testing of the diverse trip features. The staff has evaluated the licensee's November 7, 1985 submittal for this item, committing to on-line testing of the undervoltage and shunt trip devices, and has determined it to be adequate in content.

3.0 Conclusion

The staff concludes that the programs outlined in the licensee's submittal of November 7, 1983 adequately address the requirements of Generic Letter 83-28 for the areas specified in this Safety Evaluation.

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Date: October 24, 1985