

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Waterford 3 Steam Electric StationDOCKET NUMBER (2)
0 5 0 0 0 3 8 2 1 OF 0 3TITLE (4)
Reactor Trip Due To Operator DistractionEVENT DATE (5)
MONTH DAY YEAR
1 0 0 2 8 5 8 5
LER NUMBER (6)
YEAR SEQUENTIAL NUMBER REVISION NUMBER
- 0 4 2 - 0 0 1 1 0 1 8 5
REPORT DATE (7)
MONTH DAY YEAR
1 0 0 1 1 0 1 8 5
OTHER FACILITIES INVOLVED (8)
FACILITY NAMES
N/A
DOCKET NUMBER(S)
0 5 0 0 0 0OPERATING MODE (9)
1
POWER LEVEL (10)
0 1 0 1 3
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)
20.402(b) 20.405(a) 20.405(a)(1)(i) 20.405(a)(1)(ii) 20.405(a)(1)(iii) 20.405(a)(1)(iv) 20.405(a)(1)(v)
80.36(e)(1) 80.36(e)(2) 80.73(a)(2)(i) 80.73(a)(2)(ii) 80.73(a)(2)(iii) 80.73(a)(2)(iv) 80.73(a)(2)(v) 80.73(a)(2)(vi) 80.73(a)(2)(vii) 80.73(a)(2)(viii) 80.73(a)(2)(ix)
X
73.71(b) 73.71(e) OTHER (Specify in Abstract below and in Text, NRC Form 366A)LICENSEE CONTACT FOR THIS LER (12)
NAME
T. Smith, Maintenance Superintendent
TELEPHONE NUMBER
AREA CODE
5 0 4 4 6 4 - 3 1 3 8COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NPROS
CAUSE SYSTEM COMPONENT MANUFACTURER REPORTABLE TO NPROSSUPPLEMENTAL REPORT EXPECTED (14)
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At 2032 hours on October 2, 1985 Waterford 3 Steam Electric Station was at 3% reactor power when water level in Steam Generator number 2 increased to the high level reactor trip setpoint. Just prior to the trip control room operators received numerous alarms/annunciations when a short in the Emergency Diesel Generator Air Dryer motor (non-safety) caused a spike on the Static Uninterruptible Power Supply A Safety. Since the Feedwater Control System was in the manual mode of operation, the alarms/annunciations distracted the control board operator's attention long enough for the water level in Steam Generator number 2 to increase to the reactor trip setpoint.

The air dryer motor has been replaced. A Station Modification has been initiated to change the power supply for all four Emergency Diesel Generator Air Dryers. Until this modification is complete, power for all four air dryers is being temporarily supplied from a non-safety source.

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PDR ADOCK 05000382
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMR NO. 3150-0104

EXPIRES: 8/31/85

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
					0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

At 2032 hours on October 2, 1985 Waterford 3 Steam Electric Station was at 3% reactor power when the water level in Steam Generator number 2 increased to the high level reactor trip setpoint (87%). Upon receiving the reactor trip, Operations personnel immediately entered procedures OP-902-001, "Emergency Entry Procedure", and OP-902-001, "Uncomplicated Reactor Trip Recovery". Plant conditions were subsequently stabilized in mode 3 (hot standby).

Just prior to the trip control room operators received numerous alarms/annunciations when a short in the Emergency Diesel Generator Air Dryer motor (non-safety) caused a spike on the Static Uninterruptible Power Supply A Safety. The various alarms/annunciations distracted the attention of the control board operator. Therefore, the operator neglected to observe the increase in water level of Steam Generator number 2. Since the Feedwater Control System was in the manual mode of operation, the water level gradually increased until it reached the reactor trip setpoint.

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in an uncomplicated reactor trip in which no primary system parameters were exceeded. Since the Control Element Assemblies and Reactor Protective System functioned as designed, the subject event in no way placed Waterford 3 in a degraded safety condition.

CORRECTIVE ACTION

Maintenance personnel have replaced the motor for the above Emergency Diesel Generator Air Dryer. A Station Modification has been initiated to change the power supply for all four Emergency Diesel Generators Air Dryers. Until this modification is complete, power for all four air dryers is being temporarily supplied by a non-safety source.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
					OF	

TEXT: (If more space is required, use additional NRC Form 365A's) (17)

SIMILAR EVENTS

None

PLANT CONTACT

T. Smith, Maintenance Superintendent, 504/464-3138



LOUISIANA
POWER & LIGHT

142 DELARONDE STREET
P O BOX 6008 • NEW ORLEANS, LOUISIANA 70174 • (504) 366-2345

November 1, 1985

W3P85-3259
A4.05
QA

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-042-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(iv).

Very truly yours,

K.W. Cook *for*
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, J.H. Wilson, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), B.W. Churchill,
W.M. Stevenson

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