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PBL-97-0024

January 24, 1997

Document Control Desk
US NUCLEAR REGULATORY COMMISSION
Mail Station P1-137
Washington, DC 20555

Ladies/Gentlemen:

DOCKET 50-266
LICENSEE EVENT REPORT 96-014-00
STEAM GENERATOR BLOWDOWN SAMPLE
NOT PERFORMED IN ACCORDANCE WITH TECHNICAL SPECIFICATIONS
POINT BEACH NUCLEAR PLANT, UNIT 1

Enclosed is Licensee Event Report 96-014-00 for Point Beach Nuclear Plant, Unit 1. This report is provided in accordance with 10 CFR 50.73(a)(2)(i)(B), "any operation or condition prohibited by the plant's Technical Specifications." This report describes an event where the Unit 1 steam generator blowdown filter outlet sample was not obtained and analyzed in accordance with plant Technical Specifications.

Please contact us if you require additional information.

Sincerely,

A handwritten signature in cursive script, appearing to read 'Doug Johnson'.

Doug Johnson
Manager-Regulatory Services
and Licensing

280087

DAW
Enclosure

cc: NRC Resident Inspector
NRC Regional Administrator

IF22/1

9701280307 970124
PDR ADOCK 05000266
S PDR

LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH
THIS INFORMATION COLLECTION REQUEST: 50.0 HRS.
REPORTED LESSONS LEARNED ARE INCORPORATED INTO
THE LICENSING PROCESS AND FED BACK TO INDUSTRY.
FORWARD COMMENTS REGARDING BURDEN ESTIMATE
TO THE INFORMATION AND RECORDS MANAGEMENT
BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY
COMMISSION, WASHINGTON, DC 20555-0001, AND TO
THE PAPERWORK REDUCTION PROJECT

FACILITY NAME (1)

Point Beach Nuclear Plant, Unit 1

DOCKET NUMBER (2)

05000266

PAGE (3)

1 OF 4

TITLE (4)

Steam Generator Blowdown Sample Not Performed In Accordance With Technical
Specifications

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
12	29	96	96	014	00	01	24	97		05000
OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)		90	20.2201(b)		20.2203(a)(2)(v)		X		50.73(a)(2)(i)	50.73(a)(2)(viii)
			20.2203(a)(1)		20.2203(a)(3)(i)				50.73(a)(2)(ii)	50.73(a)(2)(ix)
			20.2203(a)(2)(i)		20.2203(a)(3)(ii)				50.73(a)(2)(iii)	73.71
			20.2203(a)(2)(ii)		20.2203(a)(4)				50.73(a)(2)(iv)	OTHER
			20.2203(a)(2)(iii)		50.36(c)(1)				50.73(a)(2)(v)	Specify in Abstract below
			20.2203(a)(2)(iv)		50.36(c)(2)				50.73(a)(2)(vii)	or in NRC Form 366A

LICENSEE CONTACT FOR THIS LER (12)

NAME

David Weaver

TELEPHONE NUMBER (Include Area Code)

(414) 221-3418

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES

(If yes, complete EXPECTED SUBMISSION DATE).

X

NO

EXPECTED
SUBMISSION
DATE (15)

MONTH

DAY

YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On December 29, 1996, while Point Beach Nuclear Plant (PBNP) Unit 1 was operating at 90 percent power, plant Chemistry personnel discovered that the Unit 1 steam generator blowdown filter outlet sample was not obtained and analyzed in accordance with plant Technical Specifications. Technical Specifications Table 15.7.6-1, Item 2.a, requires steam generator blowdown to be sampled and analyzed for gamma emitters twice weekly. The first weekly sample was taken on Tuesday, December 24, 1996. However, the second weekly sample, which was scheduled to occur on December 27, 1996, was missed. When discovered, the Unit 1 steam generator blowdown filter outlet was immediately sampled and analyzed satisfactorily on December 29, 1996. This event was caused by personnel error. All Technical Specifications-related chemistry procedures were reviewed to emphasize Technical Specifications-required samples. This event is being reported in accordance with the requirements of 10 CFR 50.73(a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications."

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Event Description:

On December 29, 1996, while Point Beach Nuclear Plant (PBNP) Unit 1 was operating at 90 percent power, plant Chemistry personnel discovered that the Unit 1 steam generator blowdown filter outlet sample was not obtained and analyzed in accordance with plant Technical Specifications. Technical Specifications Table 15.7.6-1, Item 2.a, requires steam generator blowdown to be sampled and analyzed for gamma emitters twice weekly. Point Beach Chemistry analytical methods Procedure CAMP-100.1 states that analyses which are required twice per week be performed at least once per four days with a minimum of two per work week. CAMP-100.1 defines the scheduled work week as Sunday through Saturday. The previous sample was analyzed on Tuesday, December 24, 1996. The second weekly sample was scheduled for the following Friday, as indicated in CAMP-101, "Daily Routine Sampling Schedule For Operating, Refueling, or Shutdown Units." However, the Friday sample was not taken.

Upon discovery, the Unit 1 steam generator blowdown filter outlet was immediately sampled and analyzed on December 29, 1996. The analysis indicated less than minimum detectable activity. During the period between samples, Unit 1 steam generator blowdown was being directed to Unit 2 service water overboard and the radiation monitoring system (RMS) was operating for monitors 1RE-219 (blowdown monitor) and 2RE-229 (service water overboard monitor). Samples of Unit 1 steam generators and Unit 2 service water overboard on December 26, 1996, indicated less than minimum detectable activity. In addition, a review of primary-to-secondary leakage data indicated that conditions did not exist to cause steam generator blowdown activity to increase during this period. All Chemistry procedures will be reviewed to emphasize Technical Specification-required samples. This event is being reported in accordance with the requirements of 10 CFR 50.73(a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications."

Cause:

The cause of this event was personnel error. Although the steam generator blowdown sample frequency and associated schedule is identified in CAMP-101, the Chemistry technician overlooked this requirement and the sample was not taken. Subsequently, the minimum Technical Specifications analysis frequency of twice per week was not met.

During evaluation of this event, Chemistry personnel identified an additional possible contributor to this event. The steam generator blowdown filter outlet (SGBFO) sample schedule is currently located in Section 1, "Secondary Chemistry," of CAMP-101. The technician responsible for the SGBFO sample is assigned to the primary-side sample routine. Hence, Section 2, "Primary Coolant," is the more appropriate section for the SGBFO sample and analysis schedule. Had the SGBFO

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sample been listed in the appropriate section of CAMP-101, the technician's awareness may have been heightened to a level where the sample would not have been missed.

Corrective Actions:

Upon discovery on December 29, 1996, the Unit 1 steam generator blowdown filter outlet (SGBFO) was sampled and analyzed. The analysis indicated less than minimum detectable activity.

A review of primary-to-secondary leakage data indicated that conditions did not exist to cause steam generator blowdown activity to increase during the period between samples.

A temporary change to CAMP-101 was issued on January 2, 1997, which relocated the SGBFO sample schedule from Section 1, "Secondary Chemistry," to Section 2, "Primary Coolant." A permanent change to CAMP-101 incorporating the relocated SGBFO sample will be completed by February 28, 1997.

All Technical Specifications-related chemistry procedures were reviewed to ensure Technical Specifications-required samples are adequately emphasized. This review was completed on January 10, 1997.

The Technician involved in this event was coached and also reviewed applicable procedures, Technical Specifications, and documentation associated with this event. The Chemistry group discussed this event at a subsequent group meeting.

The Chemistry group will evaluate the adequacy of the current method for ensuring Technical Specifications surveillance requirements are met and make appropriate recommendations by February 13, 1997.

Reportability:

This Licensee Event Report is being submitted in accordance with the requirements of 10 CFR 50.73(a)(2)(i)(B), "Any operation or condition prohibited by the plant's Technical Specifications."

Safety Assessment:

This event did not affect the operability of any safety-related equipment, function, or system. The health and safety of the public and plant personnel were not jeopardized by this event. Immediately following discovery of this condition, the Unit 1 steam generator blowdown filter outlet was immediately sampled and analyzed. The analysis indicated less than minimum detectable activity. During the period between samples, Unit 1 steam generator blowdown was being directed to Unit 2 service water overboard and the radiation monitoring system (RMS) was operating for

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monitors 1RE-219 (blowdown monitor) and 2RE-229 (service water overboard monitor). Samples of Unit 1 steam generator secondary and Unit 2 service water overboard indicated less than minimum detectable activity. In addition, a review of primary-to-secondary leakage data indicated that conditions did not exist to cause steam generator blowdown activity to increase during the period between samples.

Similar Occurrences:

The following reportable events involving missed surveillances have occurred:

<u>LER</u>	<u>Title</u>
266/96-012-00	EDG Fuel Oil System Tests Not Performed In Accordance With Technical Specifications
266/96-008-00	Missed Full Pressure Test of Containment Airlock
266/94-003-00	Reactor Coolant Sample System Containment Isolation Valve Test
266/93-006-00	Containment Isolation Valve Not Leak Tested in Accordance With Technical Specification Requirements
266/92-002-00	Missed Visual Examination of Reactor Vessel Interior
301/94-004-00	Late Reactor Coolant System Chloride Sample
301/94-002-01	Quarterly Technical Specification Test of PORV Block Valve Not Performed