

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Shoreham Nuclear Power Station Unit #1

DOCKET NUMBER (2)

0 5 0 0 0 3 2 2 1 OF 0 1

PAGE (3)

TITLE (4)

RWCU Isolation Due to erroneous differential Flow signal

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)													
0	7	1	6	8	5	8	5	0	2	7	0	0	0	8	0	9	8	5	0	5	0	0	0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)									
POWER LEVEL (10)	0 0 1	20.402(b)	20.406(c)	X	50.73(a)(2)(iv)	73.71(b)					
		20.406(a)(1)(i)	50.36(a)(1)		50.73(a)(2)(v)	73.71(e)					
		20.406(a)(1)(ii)	50.36(a)(2)		50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 356A)					
		20.406(a)(1)(iii)	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
		20.406(a)(1)(iv)	50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
		20.406(a)(1)(v)	50.73(a)(2)(iii)		50.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)

NAME
Gary G. Rhoads, Operational Compliance Engineer

TELEPHONE NUMBER

AREA CODE

5 1 6 9 2 9 - 8 3 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) ☒ NO ☐

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 16, 1985 at 0831 A.M. an erroneous High Differential Flow Signal caused the Reactor Water Cleanup Outboard Isolation valve (G33*MOV34) to isolate. The plant was in Operational Condition 2 operating at 1% power. The Reactor pressure was 85PSIG, coolant temperature was 325 Degrees F, and the Reactor Mode Switch was in Startup.

Investigation determined the cause of the erroneous signal was due to a blown fuse (G33*FN010) in the Differential Flow Instrument Loop. The fuse was replaced via a Maintenance Work Request. The associated instrument calibration was verified per SP 46.030.01 and SP 46.013.01.

Plant Management was notified of the event. There was no safety significance to the event. All plant systems operated as designed. No ECCS systems were challenged or required for the event. The NRC was notified per 10CFR50.72 at 1120 A.M.

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LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION • P.O. BOX 628 • WADING RIVER, NEW YORK 11792

TEL. (516) 929 8300

August 9, 1985

PM 85-148

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

In accordance with 10CFR50.73, enclosed is a copy of Shoreham Nuclear Power Station Unit 1's License Event Report 85-027.

Sincerely yours,

William E. Steiger, Jr.
Plant Manager

WES/gr

Enclosure

cc: Dr. Thomas E. Murley, Regional Administrator
John Berry, Senior Resident Inspector
Institute of Nuclear Power Operations, Records Center
American Nuclear Insurers

SR.A21.0200

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