U.S. NUCLEAR REGULATORY COMMISSION
REGION I

Report No. 85-56

Docket No. 50-423

License No. CPPR-113 Priority - Category C

Licensee: Northeast Nuclear Energy Company
P. O. Box 270
Hartford, Connecticut 06101

Facility Name: Millstone Nuclear Generating Station, Unit 3

Inspection At: Waterford, Connecticut

Inspection Conducted: September 30 - October 4, 1985

Inspectors: P. Clemons for — 10/29/85
J. R. White, Senior Radiation Specialist — date

P. Clemons for — 10/29/85
J. A. Cioffi, Radiation Specialist — date

Approved by: M. Shanbaky — 10/29/85
M. M. Shanbaky, Chief, PWR Radiation Safety Section — date

Inspection Summary:
Inspection on September 30 - October 4, 1985 (Report No. 50-423/85-56)

Areas Inspected: Routine, unannounced safety inspection to review the status of previously identified items, the preoperational status of the solid radioactive waste system and the task items identified in NUREG-0737; i.e., the capability for sampling and analyzing reactor coolant and containment atmosphere in post-accident conditions. The inspection involved 74 hours onsite by two region-based inspectors.

Results: Within the areas inspected, no violations were identified.

DETAILS

1.0 Personnel Contacted

D. Burglund, Vice President, Alaron Corporation
*J. Crockett, Millstone Unit 3 Superintendent, NNECO
F. DiLiberto, Engineer, SWEC
R. Enoch, Instrument and Control Group Leader, NNECO
*R. Gavensky, Radwaste QC, NNECO
*J. Kangley, Radwaste Services Supervisor, NNECO
J. Kelly, Senior Quality Assurance Engineer, SWEC
*J. LaWare, Engineering Technologist, NUSCO CQA
B. Loweth, Start-up Engineer, NNECO
*D. Miller, Start-up Manager, NNECO
D. Moore, Operations Assistant, NNECO
*L. Nadeau, Assistant Project Engineer, NUSCO
S. Orefice, Project Engineer, NUSCO
*V. Papadopoli, Supervisor, CQA, NUSCO
F. Perry, Health Physics, NNECO
*W. Rose, NNECO QC
*R. Sachetello, Health Physics, NNECO
D. Scase, Start-up Engineer, NNECO
*A. Stengel, Start-up Engineer, NNECO
R. Stotts, Operator Training Supervisor, NUSCO
*S. Turowski, ALARA Coordinator, NNECO
J. Waggoner, Training Instructor, HP, NUSCO

*Denotes attendance at the exit meeting.

2.0 Purpose

The purpose of this inspection was to review the preoperational status of the solid radioactive waste system, and of the post-accident sampling capability for containment air and reactor coolant, identified in NUREG-0737. Additional effort was expended to determine the status of previously identified items and IE bulletins and circulars.

3.0 Status of Previously Identified Items

3.1 (Closed) Circular (76-CI-16). Excessive Radiography Exposures. This circular pertained to holders of radiography licenses. No action was required by this applicant.

3.2 (Closed) Circular (80-CI-25). Radiography Events. This circular pertained to holders of radiography licenses. No action was required by this applicant.

3.3 (Open) Circular (76-CI-03) Radiation Exposure in Reactor Cavities. Details appear in paragraph 4.0 of this report.

3.4 (Open) Bulletin (78-BU-08). Radiation Levels from Fuel Element Transfer Tubes. A calculation of expected radiation levels during spent fuel transfer was performed by the Architect/Engineer for the applicant, indicating that significant radiation levels were expected based on the shielding design provided in the area between the fuel building and the containment wall. The walkway area adjacent to the Fuel Building/Containment Building interface is scheduled to be equipped with a lock and controlled as an "extreme high" radiation area. This item remains open pending completion of the installation of locked gates in this area and the institution of appropriate administrative controls.

3.5 (Closed) Circular (81-CI-07). Control of Radioactively Contaminated Material. The licensee established station procedure SHP-4917, "Unconditional Radiological Release of Material Off-site", to address the concerns raised in this circular. The procedure defines the acceptance criteria that must be met for release of material as non-radioactive, and describes acceptable detection methods and equipment sensitivity to meet the acceptance criteria. This item is considered closed.

3.6 (Open) Inspector Follow-Up (85-19-01) Status of RMS Installation and Operability. Attachment 1 describes the status of installation and operability of the Kaman Radiation Monitoring System as of October 3, 1985. Applicant representatives stated that successful completion of Phase I testing of the RMS is expected to be achieved prior to fuel load. This item will remain open pending successful completion of Phase I testing of the monitors required by the Technical Specifications.

3.7 (Open) Inspector Follow-up (85-19-02). Status of RMS procedure development. Attachment 2 describes the status of the development of procedures for the RMS. The applicant stated that approximately 50% of the procedures have been approved by the Plant Operations Review Committee (PORC), with the other 50% in process of review as of October 3, 1985. This item will remain open pending approval of the remaining procedures.

3.8 (Open) Inspector Follow-up (85-19-03). Training for health physics personnel in RMS operation. A cognizant I&C individual conducted an informal briefing of the RMS system for Health Physics personnel. A formal training program on the RMS system is under current development with the Training Department, the cognizant I&C individual, and Computer Services to provide classroom training on the RMS system to Health Physics personnel at Unit 3. This item will remain open pending successful completion of formal training for the Health Physics personnel.

3.9 (Open) Inspection Follow-up (85-19-04). Status of the start-up survey procedure. The applicant stated that the procedure for start-up survey is 98% complete. The final form of the procedure and PORC review remain. This item will remain open pending completion of the procedures and review by NRC.

3.10 (Closed) Inspection Follow-up (85-19-05). Establish requalification training for HP personnel. The inspector reviewed the 1985 requalification course for all station health physics technicians. The three-day course provided detailed instructions for I&E bulletins, circulars, and information notices of interest, current industry event releases, instrumentation, and radiobiological effects of radiation among other general Health Physics topics. Technicians were required to pass an examination with a grade of at least 80%. This item is considered closed.

3.11 (Closed) Inspector Follow-up (85-19-06). Supplies of alpha rate meters per FSAR commitments. The applicant amended Table 12.5-1 of the FSAR to correct the numbers and type of equipment available to Unit 3 and the entire Millstone site. Additionally, the description of supplies on pages 12.5-3 was amended to better reflect the status of equipment for Unit 3. The changes appeared in Amendment 15, September 1985, to the FSAR. This item is considered closed.

3.12 (Closed) Inspector Follow-up (85-19-07). Supplies of Pocket Ion Chambers (PICs) per FSAR commitments. Amendment 15, September 1985, has been submitted by the applicant to correct the numbers of PICs available to the Millstone site and Unit 3 as stated in Table 12.5-1 and 2. This item is considered closed.

3.13 (Closed) Inspector Follow-up (85-19-08). Decision to be made by applicant on use of space and facilities for Unit 3 Health Physics technical support. The applicant stated that the Units 1 and 2 Radiation Protection support facilities will be used for Unit 3 as well. These services include dosimetry issue, whole body counting, respiratory protection fit-test and issue, instrument calibrations and the decontamination facilities. This item is considered closed.

4.0 Status of High Radiation Area Control and Radiation Exposure in Reactor Cavities

The inspector toured areas of containment with the applicant and discussed the areas which would be controlled as high radiation areas. The applicant identified all areas of potential high radiation dose rates and the provisions to install locked gates to prevent access to those areas. The applicant planned to install barriers around all entries into areas of potential radiation dose. If these areas prove to be of insignificant radiation hazard during plant operation, the barriers will not be locked.

The inspector discussed control of high radiation areas with the applicant. A station procedure will be implemented, Procedure ACP 7.04, Revision 0, "Station Lock and Key Control," to control access to the keys for "high" radiation and "extreme high" radiation areas. Additionally, Operations and Health Physics personnel are currently working together to develop a specific Unit 3 procedure to control the moveable incore detectors (MIDs), stressing health physics concurrence and oversight when such activities take place.

The inspector discussed training of Health Physics technicians on the previous industry exposures incurred with the moveable incore detectors (MIDs). The Health Physics Supervisor circulated a document acknowledgment sheet (DAS) with a package of material containing a description of Unit 3's MID system and additional NRC IE circulars and notices which discussed previous industry experience of overexposures with this plant system. All Unit 3 health physics technicians signed the DAS. Additionally, the training department covered the Reactor Cavity overexposures in detail during the 1985 Health Physics technician requalification training to ensure that all health physics technicians at the Millstone Station were aware of the past history of problems with the MIDs and reactor cavity entries.

The inspector discussed the training of operations personnel on the exposure of personnel during MID activities and entry into reactor cavities. The applicant representative stated that all operations personnel were instructed on potential exposures during their Health Physics training, but that no specific issue was made of the industry experience of overexposures during MID activities and reactor cavity entries.

Circular 76-03 will remain open pending final development and approval of the operations procedure for the MIDs and reactor cavity entries and more detailed instruction of operations personnel on past industry experience with this plant system.

5.0 Post-Accident Sampling System

The applicant has installed a General Dynamics designed post-accident sampling system (PASS) to support accident sampling requirements pursuant to NUREG-0737, Item II.B.3. Testing of the system is currently being performed in accordance with T3311C, "Post-Accident Analysis Sampling." In view of engineering design control and construction deficiencies previously identified in this area relative to other Northeast Utilities nuclear operating units, the applicant appears to have developed a comprehensive test program to verify and validate that the PASS will perform as intended. An Emergency Operating Procedure currently exists for the PASS but will be modified as necessary to be commensurate with the operating parameters and characteristics determined from the preoperational test program.

While no in-depth review was performed of this system, it was noted that the ventilation fan incorporated in the sampling module to exhaust potentially contaminated air from the module during operation, was running in reverse. As a result, the sample module was being vented to the operating area. Additionally, the sample module was not provided with any drainage pathway to control valve or component leakage.

Upon completion of preoperational testing and functional verification of the PASS, a comprehensive inspection to validate PASS performance will be conducted, including corrective measures relative to the items identified during this effort. (50-423/85-56-01)

6.0 Radioactive Waste Solidification System

The applicant's radioactive waste solidification system was reviewed against descriptions and documentation identified in Attachment III to this report.

The FSAR and SER issued in 1984 describe a waste solidification process that is different than the method and equipment configuration actually intended by the applicant.

Amendment 8 to the FSAR, dated 1984, describes spent resin dewatering and liquid waste solidification processes that were eliminated from serious consideration in 1982. Reference is made to Figure 11.4-1 which was derived from the Stone and Webster Drawing No. 12179-EM-110B-2A. This drawing depicts a system which is described in the FSAR as able to solidify waste such as evaporator concentrates by connecting a portable solidification system to the installed binder transfer line, promoter and catalyst components which simultaneously blend the process chemicals into the shipping container.

Though such a binder system has been constructed as part of the solidification system, the applicant does not intend to use or maintain the system, and essentially retired the components in place in 1982. Instead, solidification of evaporator concentrates is expected to be performed by a conventional cement process which has not yet been described in the FSAR or any documentation submitted to the NRC relative to the license application. The latest SER supplement dated July 1984 still references the intent to use the original solidification process.

Additionally, on May 16, 1985, during the preoperational Phase I test program, it was identified that the installed spent resin dewatering system was incapable of dewatering to an acceptable level for shipment to a disposal site. Several Design Deficiency Reports were filed in this period which indicated that certain key components such as the Evaporator Forwarding Pump, P4, and Spent Resin Dewatering Pump, P1, were unable to function as intended by design. As a result, these components were removed from service and efforts were made to determine if the components

were essential to the dewatering process. The Test Procedure T3338-P was developed to support this effort but relied on vendor supplied equipment, i.e., NUPAC, for actual dewatering. To this end, the Phase 2 preoperational tests were conducted in an effort to demonstrate that the P4 and P1 pumps were not necessary, and that the NUPAC supplied portable dewatering system could be applied successfully. Such test performance, however, required major modification of the FSAR described system by physically removing the P4 and P1 pumps and the associated interlocks, reconfiguration of piping systems, and the application of the NUPAC procedure OM-38 which actually effect dewatering. Though such system modification was implemented by August 1985, and an FSAR change initiated internally, no information or notice of the change was ever made available for NRC review in preparation for licensing. At the time of this inspection, an FSAR change had yet to be processed.

In response to this finding, the applicant indicated that action would be initiated to update the FSAR relative to actual system configuration and processes for waste solidification and resin dewatering. This item will be reviewed in a subsequent inspection. (50-423/85-56-02)

During this inspection effort, Phase 2 testing of the system was observed. While some equipment and performance deficiencies were noted, the test demonstrated that the modified system installation, in conjunction with vendor supplied portable equipment, was able to accomplish dewatering satisfactorily. However, the tested configuration did not provide for ventilation of the fill head during resin loading, and verification that the vacuum exhaust from the NUPAC equipment could be satisfactorily handled by the plant's vent system. Additionally, the vendor procedure for dewatering, OM-38, did not provide sufficient prerequisites to assure that all alarms and system interlocks were functioning, nor did the procedure provide adequate instruction relative to alarm response.

The applicant initiated action to resolve these deficiencies by performance of system modifications as necessary for venting the fill head, arranging for verification of the ability of the vent system to accommodate the exhaust from the NUPAC system, the revision of procedure OM-38 to address prerequisites that verify system readiness (including interlocks and alarms), and the incorporation of alarm response procedures into the body of OM-38. These actions will be reviewed in a subsequent inspection (50-42385-56-03).

As part of the test program, resin dewatering was verified by removing the resin from the High Integrity Container (HIC) to qualify the presence of free-standing water. The results demonstrated that the process effectively eliminated any free-standing water. While some saturated resin remained in the bottom 2 inches of the HIC, it was apparent that the dewatering criteria of not more than 0.5% of the resin volume could be met.

Review of the Radioactive Waste Operating Procedures, which are currently implemented for Units 1 and 2, indicated that certain of the procedures will require modification or the development of additional procedures in order to reflect actual dewatering and solidification activities intended for Unit 3. For example, the procedures currently refer to a solidification method different than the cementing process intended for Unit 3. This item will be reviewed in a subsequent inspection. (50-423/85-56-04)

7. Exit Interview

The inspector met with applicant representatives (denoted in Section 1) on October 4, 1985. The inspector summarized the purpose, scope, and findings of the inspection. At no time during this inspection was written material provided to the applicant by the inspector.

ATTACHMENT I

RADIATION MONITORING SYSTEM STATUS

Instrument Number (Mark No.)	Turned Over	Field Tested	Communications Loop Established	Comments
Ventilation/Gas Monitors				
3HVRRE10A/B*				3-4 weeks before testing
3HVRRE19A/B*				3-4 weeks before testing
3HVRRE11A/B	X	X	X	
3HVRRE12A/B	X	X	X	
3HVRRE13A/B	X	X	X	
3HVRRE14A/B	X	X	X	
3HVRRE15A/B	X	X	X	
3HVRRE16A/B	X	X	X	
3HVRRE17A/B	X	X	X	
3HVRRE18A/B	X	X	X	
3HVCRE91A/B	X	X	X	
3CMSRE22A/B*	X	X	X	
3HVZRE09A/B*	X	X	X	Temporary mounting plates installed. "Unsat" - monitor A is reading 3 decades high.
3HVQRE49A/B	X	X	X	
3HVCRE16A/B*	X	X	X	

*Category IE Monitors

RADIATION MONITORING SYSTEM STATUS

Instrument Number (Mark No.)	Turned Over	Field Tested	Communications Loop Established	Comments
Liquid Process Monitors				
3CHSRE69	X	X	X	Setpoints must be established.
3CCPRE31	X	X	X	
3CNARE47	X	X	X	
3DASRE50	X	X	X	
3LWSRE70	X	X	X	Skid mounted equipment test incomplete.
3LWCRE65	X	X	X	Skid mounted equipment test incomplete.
3CNDRE07	X	X	X	
3SSRRE08	X	X	X	
3SWPRE60A/B*	X	X	X	
3GWSRE48	X	X	X	
3ARCRE21	X	X	X	"Unsat," Noise problem.
3MSSRE75	X	X	X	Communications problems.
3MSSRE76	X	X	X	Connector failed, to be replaced.
3MSSRE77	X	X	X	
3MSSRE78	X	X	X	
3MSSRE79	X	X	X	

*Category IE Monitor

RADIATION MONITORING SYSTEM STATUS

Instrument Number (Mark No.)	Turned Over	Field Tested	Communications Loop Established	Comments
		Area Monitors		
3RMSRE16	X	X	X	Switch to be replaced.
3RMSRE17	X	X	X	
3RMSRE18	X	X	X	
3RMSRE19	X	X	X	
3RMSRE20	X	X	X	
3RMSRE21	X	X	X	
3RMSRE22	X	X	X	Temporary fix, part on order.
3RMSRE24	X	X	X	
3RMSRE25	X	X	X	
3RMSRE28	X	X	X	
3RMSRE29	X	X	X	
3RMSRE31	X	X	X	
3RMSRE32	X	X	X	
3RMSRE33	X	X	X	Missing part.
3RMSRE34	X	X	X	
3RMSRE35	X	X	X	
3RMSRE36	X	X	X	
3RMSRE37	X	X	X	
3RMSRE04*	X	X	X	Detector and cable not installed.
3RMSRE05*	X	X	X	Detector and cable not installed.
3RMSRE41*	X	X	X	Detector and cable not installed.
3RMSRE42*	X	X	X	Detector and cable not installed.
3RMSRE01	X	X	X	
3RMSRE02	X	X	X	
3RMSRE03	X	X	X	
3RMSRE06	X	X	X	
3RMSRE07	X	X	X	
3RMSRE08	X	X	X	
3RMSRE09	X	X	X	
3RMSRE10	X	X	X	

RADIATION MONITORING SYSTEM STATUS

Instrument Number (Mark No.)	Turned Over	Field Tested	Communications Loop Established	Comments
Area Monitors				
3RMSRE11	X	X	X	
3RMSRE12	X	X	X	
3RMSRE13	X	X	X	
3RMSRE14	X	X	X	
3RMSRE15	X	X	X	
3RMSRE38	X	X	X	
3RMSRE52	X	X	X	

*Category IE Monitors

ATTACHMENT II

RMS SURVEILLANCE PROCEDURE SUMMARY

Procedure Number	Monitor Identification	Status/Comment
3449B01	3HVR-RE19A/B SLCRS Part/Gas Cal. Proc.	Draft
3450F01	3CND-RE07 Waste Neut. Sump Rad. Monitor Cal.	PORC approved, Rev. 0
3450J01	3LWC-RE65 Regen. Evap. Rad. Monitor Cal.	PORC approved, Rev. 0
3450K01	3SSR-RE08 S/G Blowdown Rad. Monitor Cal.	PORC approved, Rev. 0
3449J01	3HVQ-RE49 ESF Part & Gas Rad. Monitor Cal.	Draft
3449H01	3RMS-RE04/05 Cont. HR Inter. 51'4" CTMT. Cal. Proc.	Draft
3449G01	3HVC-RE16A/B Control Bldg. Inlet. Cal. Proc.	Draft
3449A11	3HVR-RE10A/B Vent Part/Gas 66'6" Aux. Bldg. Oper. Test	Draft
3449B11	3HVR-RE19A/B SLCRS Part/Gas Oper. Test	Draft
3449D11	3SWP-RE60A/B Cont. Recirc. Cool Outlet Oper. Test	Cancelled
3449E11	3RMS-RE41/42 Fuel Drop Monitor 51'4" Oper. Test	Draft
3449G11	3HVC-RE16A/B Control Bldg. Inlet Oper. Test	PORC approved, Rev. 0
3449H11	3RMS-[illegible]'05 Cont. HR Inter. 51'4" CTMT. Oper. Test	PORC approved, Rev. 0
3450F11	3CND-RE07 Waste Neut. Sump Rad. Monitor Oper. Test	PORC approved, Rev. 0
3450G11	3DAS-RE50 Turb. Weldg. Floor Drains Rad. Monitor Oper. Test	PORC approved, Rev. 0
3450H11	3LWS-RE70 Liquid Waste Rad. Monitor Oper. Test	PORC approved, Rev. 0
3450J11	3LWC-RE65 Regen. Evap. Rad. Monitor Oper. Test	PORC approved, Rev. 0
3450K11	3SSR-RE08 S/G Blowdown Rad. Monitor Oper. Test	Draft
3449A01	3HVR-RE10A/B Vent Part/Gas 66'6" Aux. Bldg. Cal. Proc.	Draft
3449C01	3CMS-RE22A/B CTMT Atmos. Part/Gas Cal. Proc.	Draft
3449C11	3CMS-RE22A/B CTMT Atmos. Part/Gas Oper. Test	Draft
3449D01	3SWP-RE60A/B TM Recirc. Cool Outlet Cal. Proc.	Cancelled
3449E01	3RMS-RE41/42 Fuel Drop Monitor 51'4" Cal. Proc.	Draft
3449E21	Fuel Pool Storage Criticality Monitor	PORC approved, Rev. 1
3449E31	Fuel Pool Storage Crit. Monitor Oper. Test	PORC approved, Rev. 0
3449F01	3HVZ-RE09A/B Recom. Cub. A/B Exh. 7'6" Cal. Proc.	Cancelled
3449F11	3HVZ-RE09A/B Recom. Cub. A/B Exh. 7'6" Oper. Test	Cancelled

RMS SURVEILLANCE PROCEDURE SUMMARY

Procedure Number	Monitor Identification	Status/Comment
3449J11	3HVQ-RE49 ESF Part & Gas Rad. Monitor Oper. Test	Draft
3450D01	3MSS-RE75, 76, 77, 78 Main Stm. Rel. Line A-D Cal. Proc.	Draft
3450D11	3MSS-RE75, 76, 77, 78 Main Stm. Rel. Line A-D Oper. Test	PORC approved, Rev. 0
3450E01	3MMS-RE79 Turb. Drv. Aux. Fdw. Pump Dsch. Cal. Proc.	Draft
3450E11	3MMS-RE79 Turb. Drv. Aux. Fdw. Pump Dsch. Oper. Test	PORC approved, Rev. 0
3450G01	3DAS-RE50 Turb. Bldg. Floor Drains Rad. Monitor Cal.	PORC approved, Rev. 0
3450H01	3LWS-RE70 Liquid Waste Rad. Monitor Cal.	PORC approved, Rev. 0

ATTACHMENT III

RMS GENERIC PROCEDURE SUMMARY

Generic No. (GPIC)	I&C Procedure No.	Monitor Identification	Status/Comment
20.01	3490B01	Kaman Science (KMG-HRN) Radiation Monitor	Approved,* Rev. 0
20.02	3490B02	Kaman Science (KMG-HRH) Radiation Monitor	Draft
20.03	3490B03	Kaman Science (KMPG) Radiation Monitor	Approved,* Rev. 0
20.04	3490B04	Kaman Science (KML) Radiation Monitor	Approved,* Rev. 1
20.05	3490B05	Kaman Science (KMG) Radiation Monitor	Draft
20.06	3490B06	Kaman Science Hydrogen Vent Monitor	Draft
20.07	3490B07	Kaman Science Hydrogen Recombiner Monitor	Deleted
20.08	3490B08	Kaman Science (KMA-HR) Radiation Monitor	Approved,* Rev. 0
20.09	3490B09	Kaman Science (KMA-MR) Radiation Monitor	Deleted
20.10	3490B16	Kaman Science (KMA) Radiation Monitor	Approved,* Rev. 0
20.11	3490B11	Kaman Science Fuel Failure Monitor	In Review
20.12	3490B12	Kaman Science Service Water Effluent Monitor	In Review
20.13	EC1-4	Eberline Radiation Monitor	In Development

*Approval of these procedures is by the I&C Department Supervisor and the Startup Manager.

Radioactive Waste Operating Procedures

Number	Title	Rev.	Date
RW 6001/26001/36001	Waste Classification Implementation Program	1	8/1/85
RW 6002/26002/36002	Determination of the Waste Classification for Radioactive Waste Offered for Shallow Land Burial	3	10/1/85
RW 6003/26003/36003	Radioactive Materials Shipping Compliance	1	8/1/85
RW 6004/26004/36004	Shipment of Radwastes	2	8/1/85
RW 6005/26005/36005	General Radioactive Materials Shipment	1	8/1/85
RW 6006/26006/36006	Receipt of Radioactive Material	1	5/31/85
RW 6007/26007/36007	Container Control and Accountability	1	8/1/85
RW 6008/26008/36008	Drum Compacting Procedure	2	8/1/85
RW 6009/26009/36009	Radwaste Shipment Survey Procedure	1	8/1/85
RW 6010/26010/36010	Loading of Chem-Nuc Shipment Cask	1	8/1/85
RW 6011/26011/36011	Operating the Mobile Shield Assembly (Atcor Cask)	1	8/1/85
RW 6012/26012/36012	Packing Noncompactable LSA Containers	2	8/1/85

ATTACHMENT III

MNPS-3 Final Safety Analyses Report (FSAR), Section 11.4, Solid Waste Management, Amendment 8, dated May 1984

Safety Evaluation Report (SER) Supplement dated July 1984

Operating Procedure OP3338A, Revision 0, "Radioactive Solid Waste," dated September 19, 1985

Test Procedure T3338-P, Revision 0, "Radioactive Solid Waste System," dated August 15, 1985

Vendor Procedure OM-38, "Nuclear Packaging Dewatering System," Revision 1, dated September 26, 1985

Stone and Webster Drawing No. 12179-EM-110B-2A, "Piping and Instrument Diagram - Radioactive Solid Waste," dated September 17, 1982

Licensee Response to NRC Interrogatory No. Q460.17

NRC Standard Review Plan 11.4, Solid Waste Management Systems