

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) VERMONT YANKEE NUCLEAR POWER STATION	DOCKET NUMBER (2) 0 5 0 0 0 2 7 1	PAGE (3) 1 OF 0 3
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TITLE (4)  
1985 Appendix J Type B and C Testing

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)												
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)										
0	9	2	6	8	5	8	5	0	0	7	0	0	1	0	2	3	8	5			0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more of the following) (11)									
POWER LEVEL (10) 0 1 0 0	20.402(b)	20.408(e)	80.73(a)(2)(iv)	73.71(b)							
	20.408(a)(1)(i)	80.36(a)(1)	80.73(a)(2)(v)	73.71(e)							
	20.408(a)(1)(ii)	80.36(a)(2)	80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)							
	20.408(a)(1)(iii)	80.73(a)(2)(i)	80.73(a)(2)(vii)(A)								
	20.408(a)(1)(iv)	80.73(a)(2)(ii)	80.73(a)(2)(vii)(B)								
	20.408(a)(1)(v)	80.73(a)(2)(iii)	80.73(a)(2)(x)								

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME	AREA CODE		
James P. Pelletier, Plant Manager	8 0 2 2 5 7	7 7 1 1	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	
X	S	B	V R 3 4 0	Y	X	S	J	V A 3 9 1	Y	
X	S	B	V W 0 3 0	Y	X	A	A	V L 6 3 1	Y	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO		0 6	1 5	8 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing Type C Leak Rate Testing, MSIV-86C, MSD-77, FDW 96A, CRD-412A, CRD-413A, CRD-413B, CA-89B, RWCU-15 and CRD-181 were found to have seat leakage above that permitted by Tech. Spec. section 3.7.A.4. This resulted in the total Appendix J Type B and C limit of 14.75 lbm/hr being exceeded which does not meet Tech. Spec. section 3.7.A.3 requirement.

Vermont Yankee will perform maintenance on all of the above valves to determine cause of failure and retest them to ensure that seat leakage is within Tech. Spec. allowables prior to plant startup following the 1985/86 refueling outage.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
VERMONT YANKEE NUCLEAR POWER STATION	0 5 0 0 0 2 7 1 8 5	—	0 0 7	— 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 365A's) (17)

While performing Type C Leak rate testing MSIV-86C, MSD-77, FDW-96A, CRD-412A, CRD-413A, CRD-413B, CA-89B, RWCU-15 and CRD-181 were found to have seat leakage above that permitted by Tech. Spec. section 3.7.A.4. (RWCU-15 and CRD-181 are included in the B and C Testing for operability status and good maintenance practice but are not in VY's NRC Approved Appendix J program.

For MSIV-86C, MSD-77, CRD-413B, CA-89B, RWCU-15 and CRD-181 a second isolation valve in the applicable system was tested and met the acceptance criteria.

For penetration X-31F both isolation valves CRD-412A and CRD-413A exceeded the single valve acceptance criteria of 0.522 lbm/hr. For CRD-412A the measured leakage was 1.52 lbm/hr and for CRD-413A it was 1.912 lbm/hr.

For penetration X-9B VY was unable to pressurize to the test condition for the out-board check valve FDW-96A. (The inboard feedwater check valves are exempt from appendix J testing requirements). Due to this, the leakage was assumed higher than the total allowable Type B and C limit of 14.75 lbm/hr.

VY is currently developing a program to determine why FDW-96A, a valve installed to meet Appendix J criteria, has failed 3 consecutive leakage tests and a similar valve in the other feedwater line has consistently passed. This program will consider the following:

1. Seating material
2. Operational differences between the feedwater lines
3. Valve geometry/alignment differences
4. Temperature and flow considerations

Vermont Yankee will perform maintenance on all of the above valves and retest them to ensure the seat leakage is within Tech. Spec. allowables prior to plant startup.

No similar events have been reported on MSIV-86C, CRD-413A, CRD-413B and CA-89B in the last 5 years.

A similar event was reported on CRD-412A as LER 84-11.

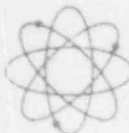
Similar events were reported on FDW-96A as LER 83-10 and LER 84-11.

**LICENSEE EVENT REPORT (LER) FAILURE CONTINUATION**

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
VERMONT YANKEE NUCLEAR POWER STATION	0500027185-007-0003	OF	03		

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

[illegible]



# VERMONT YANKEE NUCLEAR POWER CORPORATION

P. O. BOX 157  
GOVERNOR HUNT ROAD  
VERNON, VERMONT 05354

October 28, 1985

VYV85-467

U.S. Nuclear Regulatory Commission  
Document No. 50-271  
Washington, D.C. 20555

REFERENCE: Operating License DPR-28  
Docket No. 50-271  
Reportable Occurrence No. LER 85-07

Dear Sirs:

As defined by 10CFR50.73, we are reporting the attached Reportable Occurrence as LER 85-07.

Very truly yours,

VERMONT YANKEE NUCLEAR POWER CORPORATION

James P. Pelletier  
Plant Manager

HMM/drc

cc: Regional Administrator  
USNRC Office of Inspection and Enforcement  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

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