

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Browns Ferry - Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 2 5 9 1 OF 0 2

PAGE (3)

TITLE (4)

Secondary Containment Isolation From A High Radiation Alarm

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)				
									Browns Ferry - Unit 2	0 5 0 0 0 2 6 0				
0 7	1 0	8 5	8 5	0 3 5	0 0 0	8 0	6 8	5	Browns Ferry - Unit 3	0 5 0 0 0 2 9 6				
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)											
N			20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)		
POWER LEVEL (10)			20.405(a)(1)(i)			50.36(e)(1)			50.73(a)(2)(v)			73.71(c)		
0 0 0			20.405(a)(1)(ii)			50.36(e)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
			20.405(a)(1)(iii)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(A)					
			20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)					
			20.405(a)(1)(v)			50.73(a)(2)(ii)			50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)

NAME  
David L. Smith

TELEPHONE NUMBER

AREA CODE

2 1 0 5 7 1 2 9 1 - 1 3 8 1 6 1 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) ☒ NO ☐

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

During refueling operations on unit 1, a high radiation alarm on the refuel floor was experienced due to radiation from the moisture separator as it was being moved from the vessel cavity to the equipment pit. This is an expected occurrence; however, there were no written procedures to alert the licensed reactor operator and the event is, therefore, categorized for reporting as an inadvertent engineered safety feature initiation. Operating and maintenance instructions for vessel internals disassembly are being appropriately revised to address the possibility of this isolation function.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)  Browns Ferry - Unit 1	DOCKET NUMBER (2)  0500025985	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		85	035	00	2	OF	02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Units 1 and 2 were in refueling outages, and unit 3 was in an extended maintenance outage.

During normal reactor (RCT) disassembly at 1455 on July 10, 1985, part of the moisture separator (MSR) was momentarily lifted out of the reactor vessel cavity water in order to transfer to the storage pit. This caused reactor building ventilation radiation monitor (IL) (RM 90-143) to alarm and initiated secondary containment (BD) isolation. As the moisture separator was once again totally submerged, the radiation monitors returned normal readings. No unplanned personnel radiation exposures occurred.

The required safety system actuations were received as designed (refuel zone isolation, unit 1 reactor zone isolation, partial primary containment isolation system (JM) isolation, standby gas treatment (VA) initiation, and control room emergency ventilation (VI) initiation). After verification that no abnormal situation existed, the licensed reactor operator returned the affected safety systems to standby readiness.

Reactor disassembly procedures did not specifically address that the transfer of the moisture separator could trip the subject radiation monitor and cause a secondary containment isolation. This event is, therefore, categorized as an inadvertent engineered safety feature initiation. Operating and maintenance instructions are being revised to include notification requirements to alert the unit control room operators prior to raising the moisture separator for reactor assembly/disassembly.

Responsible Plant Section - N/A

Previous Events - None

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant  
P. O. Box 2000  
Decatur, Alabama 35602

August 6, 1985

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

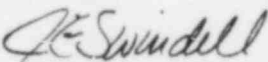
Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 1 -  
DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE  
OCCURRENCE REPORT BFRO-50-259/85035

The enclosed report provides details concerning secondary containment  
isolation from a high radiation alarm. This report is submitted in  
accordance with 10 CFR 50.73(a)(2)(v).

Very truly yours,

TENNESSEE VALLEY AUTHORITY



Robert L. Lewis  
Acting Plant Manager  
Browns Ferry Nuclear Plant

Enclosures

cc (Enclosures):

Regional Administrator  
U. S. Nuclear Regulatory Commission  
Office of Inspection and Enforcement  
Region II  
101 Marietta Street, Suite 2900  
Atlanta, Georgia 30303

INPO Records Center  
Suite 1500  
1100 Circle 75 Parkway  
Atlanta, Georgia 30339

NRC Resident Inspector, BFN

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