

# CERTIFIED

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ACRS SUBCOMMITTEE MEETING MINUTES ON  
SAFETY PHILOSOPHY, TECHNOLOGY, AND CRITERIA  
OCTOBER 9, 1985  
WASHINGTON, DC

The ACRS Subcommittee on Safety Philosophy, Technology, and Criteria met on October 9, 1985 at 1717 H Street, N.W., Washington, D.C. The purpose of this meeting was to discuss the status of the work performed by the NRC Staff in developing containment performance guidelines. The Subcommittee heard presentations from representatives of RES and NRR. The meeting was began at 3:00pm and adjourned at 5:00pm. The principle attendees were:

D. Okrent, Subcommittee Chairman  
J. Ebersole, Member  
F. Remick, Member  
C. Wylie, Member  
W. Kerr, Member  
C. Michelson, Member  
R. Savio, ACRS Staff  
M. Ernst, RES Staff  
Z. Rosztoczy, NRR Staff  
J. Rosenthal, NRR Staff

## Highlights

1. The NRC Staff has been directed by Chairman Palladino to develop containment performance design objectives for incorporation into a Safety Goal Policy. The NRC Staff intends to develop proposed objectives and, after Commission approval, to seek public comment and publish revised guidelines. The design objectives would be then evaluated over a trial use and implementation period. At the end of the trial use, the design objectives would be reevaluated and final recommendations made to the Commission. The proposed budgeting is as follows:

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	<u>PSY</u>	<u>T/A</u>	
FY 86	3.5	275K	
FY 87	4.0	400K	
FY 88	4.0	275K	} Proposed use and implementation period
FY 89	4.0	175K	

Issuance of the containment performance design objectives and implementation guidelines is scheduled for September 1987. A copy of the NRC Staff plan for achieving this is included as Attachment A.

3. The NRC Staff is just beginning the work containment performance design objectives and is not yet recommending any specific guidelines. Some of the alternatives being considered are:
  - (a) The proposal put forth in NUREG-0739 ("A Approach to Quantitative Safety Goals for Nuclear Power Plants," October 1980) specifying an operating limit of 0.1 conditional probability and a safety goal of 0.01 conditional probability for containment integrity given a severe accident.
  - (b) Requiring some specified reduction in the released fission products via containment performance.
  - (c) Establishing an acceptable probability for a large scale release and a minimum level of containment performance and then allowing tradeoff between core melt frequency and containment failure in achieving this probability for a large scale release.
3. It would be the NRC Staff's intention to develop a preliminary performance objective and to evaluate this performance objective through trial use. Development and evaluation of this goal would

relay heavily on reference plant analysis. The NRC Staff could relay significantly on the following work:

- (a) NUREG-0956, "Reassessment of the Technical Bases for Estimating Source Terms," draft report for comment.
- (b) NUREG-1037, "Containment Performance Working Group Report," draft report for comment.
- (c) NUREG-1079, "Estimates of Early Containment Loads from Core Melt Accidents," draft.
- (d) SARRP Reports and the subsequent draft of NUREG-1150.
- (e) Past PRAs and their reviews with emphasis on Indian Point/Zion, Limerick, Millstone 3, GESSAR.
- (f) Seismic Studies completed to date.
- (g) Safety Goal Evaluation Steering Group Report.
- (h) PRA Reference Document (NUREG-1050).

It was noted that a useful concept to be considered in this process was the use of a containment performance design objective which was dependent on the class of site under consideration. It was additionally noted that containment performance may well be very dependent on plant specific details of the containment design. This could limit the usefulness of the conclusions drawn from reference plant analysis.

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NOTE: Additional meeting details can be obtained from a transcript of this meeting available in the NRC Public Document Room, 1717 H Street, N.W., Washington, D.C., or can be purchased from ACE-Federal Reports, 444 North Capitol Street, Washington, D.C. 20001, (202) 347-3700.