NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Washington Nuclear Plant - Unit 2	0 5 0 0 0 3 9 7	1 OF 0 2

TITLE (4)
Surveillance Testing During Single Loop Operation

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 7	2 0	8 5	8 5	—	0 4 9	—	0 0	0 8	0 9	8 5		0 5 0 0 0
												0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
1		20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 5 6		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
W. S. Davison, Compliance Engineer	5 0 9 3 7 7 1 - 1 2 5 0 1 Ext. 2726

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While increasing power above 50% of rated core thermal power in single recirculation loop operation on 7/20/85, core pressure drop baseline data was not obtained as required by Technical Specification 3.4.1.1.

As a consequence of this, it was necessary to reduce power to the thermal power limit specified in the Technical Specifications and establish the baseline data prior to again exceeding the thermal power limit (as required by Technical Specifications).

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant Conditions

a) Power Level - 56%
b) Reactor Mode - 1

Event

Through the issuance of Technical Specification 3.3.10 and major revisions to 3.4.1.1 on 7/19/85, the Plant was allowed to increase power above 50% rated core thermal power with one reactor coolant system recirculation loop not in operation. As a part of these Technical Specification changes, new surveillance requirements were imposed which require the accumulation of baseline neutron instrumentation noise and core pressure drop noise data prior to exceeding a predetermined limit of thermal power at specified flow conditions.

Baseline data was obtained under increasing flow conditions as power was increased above 50% on 7/19 and 7/20; however, data for core pressure drop was inadvertently dropped from the Plant data acquisition system. Consequently the established baseline was inadequate and it was necessary to reduce power from 56% to 52% (back to the maximum power limit curve) and obtain baseline data for core pressure drop. Reactor power was subsequently increased above the power limit curve and the 8 hour surveillance requirements, at the elevated power level, have since been satisfied.

Immediate Corrective Action

o When it was discovered that the baseline data was inadequate, power was immediately reduced to the thermal power limit at the existing flow condition and baseline data was reestablished prior to further power escalation.

Further Corrective Action

No further corrective action was deemed necessary. This surveillance has been performed successfully on an eight hour frequency since the incident. This event is considered to be an isolated one due to personnel error and does not represent a programmatic deficiency.

Safety Significance

This event did not affect the safety of the public or that of Plant personnel. Upon obtaining a complete set of baseline data, all values were found to be within acceptable limits. At no point was the Plant operated outside of expected parameters.

Similar Events

None

EIIS Information

NA

Washington Public Power Supply System
P.O. Box 968 3000 George Washington Way Richland, Washington 99352 (509) 372-5000

Docket No. 50-397

August 9, 1985

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: NUCLEAR PLANT NO. 2
LICENSEE EVENT REPORT NO. 85-049

Dear Sir:

Transmitted herewith is Licensee Event Report No. 85-049 for WNP-2 Plant. This report is submitted in response to the report requirements of 10CFR50.73 and discusses the item of reportability, corrective action taken, and action taken to preclude recurrence.

Very truly yours,

JW Baker for

C.M. Powers (M/D 927M)
WNP-2 Plant Manager

CMP:la

Enclosure:
Licensee Event Report No. 85-049

cc: Mr. John B. Martin, NRC - Region V
Mr. A. D. Toth, NRC - Site (901A)
Ms. Dottie Sherman, ANI
INPO Records Center - Atlanta, GA

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