

## LICENSEE EVENT REPORT (LER)

(See reverse for required number of  
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY  
INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS  
LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED  
BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN  
ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T.  
6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC  
20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104),  
OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

Millstone Nuclear Power Station Unit 1

DOCKET NUMBER (2)

05000245

PAGE (3)

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TITLE (4)

Previous Unit Director Did Not Meet Technical Specification Requirements

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
12	20	96	96	064	00	01	20	97	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)							
POWER LEVEL (10)		000	20.2201(b)		20.2203(a)(2)(v)		<input checked="" type="checkbox"/>		50.73(a)(2)(i)	50.73(a)(2)(viii)
			20.2203(a)(1)		20.2203(a)(3)(i)				50.73(a)(2)(ii)	50.73(a)(2)(x)
			20.2203(a)(2)(i)		20.2203(a)(3)(ii)				50.73(a)(2)(iii)	73.71
			20.2203(a)(2)(ii)		20.2203(a)(4)				50.73(a)(2)(iv)	OTHER
			20.2203(a)(2)(iii)		50.36(c)(1)				50.73(a)(2)(v)	Specify in Abstract below or in NRC Form 366A
			20.2203(a)(2)(iv)		50.36(c)(2)				50.73(a)(2)(vii)	

## LICENSEE CONTACT FOR THIS LER (12)

NAME

Robert W. Walpole, MP1 Nuclear Licensing Manager

TELEPHONE NUMBER (Include Area Code)

(860)440-2191

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

## SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE).	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION	MONTH	DAY	YEAR
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## ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On December 20, 1996, with the plant in the COLD SHUTDOWN condition, an assessment of the training and qualifications of the previous Unit Director for Millstone Unit No. 1 identified a violation of Technical Specification (TS) Section 6.3.1, Training. TS 6.3.1 requires the facility staff to meet or exceed the requirements of ANSI N18.1-1971 for comparable positions. The standard requires the Plant Manager (i.e. Unit Director) to have the experience and training normally required for examination for a Senior Reactor Operator (SRO) license or to designate an alternate with the proper qualifications. The assessment of the former Unit Director qualifications found no documentation of a training evaluation. Additionally, the individual did not have a designated alternate. This event was caused by a lack of procedure compliance. There were no actual safety consequences as a result of this event. The safety significance of this event is that the Unit Director did not satisfy the requirements of ANSI N18.1. The individual was not qualified to review and approve changes to the facility design or procedures based on the conditions and limitations of the license for the unit.

As part of the corrective actions, the qualifications of the current Director, Unit Operations has been satisfactorily documented. Other corrective actions will include documentation of other Key Personnel and verification of the accuracy of the documents approved by the former Unit Director at Plant Operations Review Committee and Site Operations Review Committee meetings.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

I. Description of Event

On December 20, 1996, with the plant in a COLD SHUTDOWN condition, an assessment of the former Unit Director's training and qualifications revealed that the TS Section 6.3.1 was violated. This assessment was the result of the NRC Senior Resident Inspector's request for documentation to support the qualifications of the former Unit Director. The assessment, completed on December 20, 1996, determined that the records of an assessment of previous equivalent SRO training of the former director could not be located.

Procedure OA 1, "Organization, Responsibilities, and Assumption of Responsibilities for Key Site Personnel," is based on the license requirements of TS Section 6, "Administrative Controls." OA 1 identifies the training requirements for the Unit Director as 'Must hold a Senior Operator's License on his respective Unit or possess knowledge equivalent to that required to obtain a Senior Operator's License.'

Procedure TQ-1, Training and Qualification, specifies the controls for the Nuclear Training Department (NTD) training programs including the SRO training program. The Nuclear Training Manual, referenced by TQ-1, includes instructions for validation and documentation of equivalent knowledge.

The documentation requirements of procedure TQ-1 were not met. Specifically, the records required by procedure TQ-1 to document that the Unit Director had equivalent SRO knowledge do not exist.

II. Cause of Event

The cause of this event was failure to comply with the approved procedures.

On August 24, 1995, an assessment by the Director, Quality and Assessment Services (QAS) of the former Unit Director's qualifications missed an opportunity to identify the lack of procedure compliance. The assessment stated that "A review of the qualifications of the new Director when compared to the requirements defined in ANSI N18.1, did not note any abnormalities."

III. Analysis of Event

This event is reportable pursuant to 10CFR50.73(a)(2)(i)(B), any operation or condition prohibited by the plant's Technical Specifications. The procedures for establishing and documenting personnel qualification and training are adequate, but were not followed.

There were no actual safety consequences as a result of this event. The safety significance of this event is that the former Unit Director did not satisfy the requirements of ANSI N18.1. The individual was not qualified to review and approve changes to the facility design or procedures based on the conditions and limitations of the license for the unit.

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

IV. Corrective Actions

- The qualifications of the current Millstone Unit No. 1 Director, Unit Operations has been satisfactorily documented.
- Northeast Nuclear Energy Company (NNECO) commits to verify and document the qualifications of other Key Personnel by January 31, 1997.
- NNECO commits to verify the accuracy of the documents approved by the former Unit Director at Plant Operations Review Committee and Site Operations Review Committee meetings. This verification will be conducted by grouping the documents by the systems that they affect and then completing the verification prior to declaring the individual systems operable.
- Additionally, for the systems that are currently operable, NNECO commits to complete the verification by February 28, 1997.

V. Additional Information

This event and the corrective actions will be shared with Millstone Unit Nos. 2 and 3 for their applicability.

Similar Events

None

Manufacturer Data

Not Applicable