

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Rancho Seco Nuclear Generating Station, Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 1 2 1 OF 0 2

TITLE (4)

RPS Channel Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)						
0	6	1	2	8	5	8	5	0	1	1	0	5	0	0	0		
									NONE		0	5	0	0	0		

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)											
POWER LEVEL (10) 0 0 1 0	20.402(b)			20.406(e)			<input checked="" type="checkbox"/> 50.73(a)(2)(iv)			73.71(b)		
	20.406(a)(1)(i)			50.36(e)(1)			50.73(a)(2)(v)			73.71(a)		
	20.406(a)(1)(ii)			50.36(e)(2)			50.73(a)(2)(vii)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
	20.406(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)					
	20.406(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)					
20.406(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)						

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Ron W. Colombo, Regulatory Compliance Supervisor	AREA CODE 9 1 6 4 5 2 - 3 2 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input checked="" type="checkbox"/>	<input type="checkbox"/>				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 12, 1985, it was reported via an internal Occurrence Description Report that Reactor Protection System (RPS) Channels "C" and "D" tripped on the 1820 psig Shutdown Bypass high pressure setpoint during plant heatup. The event occurred at 1402 hours on June 11, 1985 with three (3) reactor coolant pumps running and control rod safety group 1 withdrawn 100%.

A review of the event revealed that the B.2 procedure, "Plant Heatup and Startup", instructs the operator to perform specified operations when the reactor coolant pressure approaches 1820 psig. This instruction, combined with the conservatism allowed for the associated instrumentation errors, is inadequate to assure that the Shutdown Bypass high pressure trip setpoint is not inadvertently exceeded, thus tripping the RPS channels.

As a corrective action taken to prevent a recurrence of this event, the B.2 procedure will be revised to instruct the operator to perform the specified operations as the pressure approaches 1700 psig, allowing sufficient margin to prevent the inadvertent actuation of Bypass Shutdown high pressure trip. This action will be completed prior to the next reactor coolant system heatup.

There were no effects on plant or public safety as a result of this event.

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PDR ADDCK 05000312
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Rancho Seco Nuclear Generating Station, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 2	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	— 0 1 1 1	— 0 1 0	0 1 2	OF 0 1 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On June 12, 1985, it was reported via an internal Occurrence Description Report that Reactor Protection System (RPS) Channels "C" and "D" tripped on the 1820 psig Shutdown Bypass high pressure setpoint during reactor coolant system heatup to hot shutdown conditions. The event occurred at 1402 hours on June 11, 1985 with three (3) reactor coolant pumps running and control rod Safety Group 1 withdrawn 100%. At this time, operation of the plant was in accordance with procedure B.2, "Plant Heatup and Startup".

Each RPS channel is provided with a key operated Shutdown Bypass switch to permit control rod drive testing when the reactor is shutdown and the reactor coolant system is depressurized below the low pressure trip point of 1900 psig. Associated with each Shutdown Bypass switch is a high pressure bistable, set to open (trip) its contacts at 1820 psig increasing RCS pressure. The purpose of the 1820 psig Shutdown Bypass high pressure trip is to prevent normal operations with an RPS channel bypassed.

A review of the event revealed that Section 4.3.17 of the B.2 procedure instructs the operator to "Perform the following when reactor coolant pressure approaches the high pressure trip point, 1820 psig". Operations personnel stated that normally, at this point in heatup, the operator would be monitoring reactor coolant pressure by means of the wide range (0-2500 psi) instrumentation. The smallest increment on this instrumentation represents 50 psi. The instruction, combined with the conservatism allowed for the associated instrumentation errors, is inadequate to assure that sufficient margin between the reactor coolant pressure and the Shutdown Bypass is maintained to prevent an inadvertent trip of the RPS Channels.

The corrective action taken to prevent a recurrence of this event will be to revise Section 4.3.17 to instruct the operator to perform the specified operations as the pressure approaches 1700 psig, thus allowing sufficient margin to prevent the inadvertent actuation of the Shutdown Bypass high pressure trip. This corrective action will be completed prior to the next reactor coolant system heatup.

A Root Cause Analysis will be performed to investigate the sequence of events leading to the trip.

If the Root Cause Analysis conclusions differ from those of the present investigation, a supplement to this report detailing the conclusions and further corrective actions, if any, will be submitted.

There were no effects on plant or public safety as a result of this event.



SMUD

SACRAMENTO MUNICIPAL UTILITY DISTRICT ☐ 6201 S Street, P.O. Box 15830, Sacramento CA 95852-1830, (916) 452-3211
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July 17, 1985

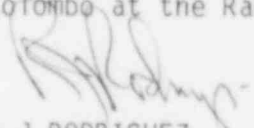
REGION V IAF

J B MARTIN, REGIONAL ADMINISTRATOR
REGION V OFFICE OF INSPECTION AND ENFORCEMENT
ATTN: DOCUMENT CONTROL DESK
U S NUCLEAR REGULATORY COMMISSION
WASHINGTON DC 20555

DOCKET NO. 50-312
LICENSE NO. DPR-54
LICENSEE EVENT REPORT NUMBER 85-11

In accordance with the requirements of 10 CFR 50.73(a)(2)(iv), the Sacramento Municipal Utility District hereby submits Licensee Event Report Number 85-11.

If there are any questions concerning this report, please contact Mr. Ron Colombo at the Rancho Seco Nuclear Generating Station.


R J RODRIGUEZ
ASSISTANT GENERAL MANAGER,
NUCLEAR

Attachment

cc: Region V (2)
INPO

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IE-29