

APPROVED OMB NO. 3150-0104
EXPIRES 8/31/95

ABSTRACT (Limit to 1400 spaces i.e. approximately fifteen single-space typewritten lines) (18)

CV-4025, the inner isolation valve was operable and capable of performing its isolation function during the event.

1622

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1): BIG ROCK POINT PLANT	DOCKET NUMBER (2): 0 5 0 0 0 1 5 5	LER NUMBER (8):			PAGE (3):		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	0 0 7	0 0	0 2	OF	0 3

TEXT (if more space is required, use additional NRC Form 365A's) (17)

Description

On 09/18/85 (0500 hours) while unit was in a scheduled refueling outage, the containment dirty sump isolation valves CV-4025 (inner) and CV-4103 (outer) were closed and tagged to allow maintenance procedure MGP-9, diaphragm replacement on CV-4103. During discussions with Repairman prior to removal from service, the Shift Supervisor was led to believe that diaphragm replacement would not affect valve travel or stem adjustment. On this basis and the fact that the valve closes on air failure, the Shift Supervisor concluded that subsequent testing was not required and that tagging the valves closed during diaphragm replacement was sufficient to maintain containment integrity.

Following diaphragm replacement, on 09/19/85 the valve was successfully stroke tested to verify diaphragm operability and reopened at 2015 hours, thus returning the dirty sump isolation line to service.

On 09/24/85, during review of completed maintenance activities (CV-4103 diaphragm replacement), a question was raised whether the activity had the potential for affecting the "leak-tightness" of the valve and the need to perform a component leak rate test to verify operability. Investigation showed that the diaphragm is threaded onto the stem and is held in position by a lock nut. When the new diaphragm is installed the potential exists to misposition the lock nut and affect operability. On this basis, CV-4025 was closed and CAUTION tagged at 1800 hours to insure containment integrity until leak rate testing could be performed.

On 09/25/85 (1100 hours), further investigation showed that during the time period from 09/19/85 (2015 hours) until 09/24/85 (1800 hours) in which valves CV-4103 and CV-4025 were open, fuel bundles were moved within the spent fuel pool on 10 occasions in violation of System Operating Procedure 44 which requires containment integrity to move fuel.

Since the event caused one of two containment isolation valves to become inoperable during a mode which requires containment integrity (fuel moves in containment), reportability per 10 CFR 50.73 (a)(2)(vii)(D) was concluded.

Cause

Inadequate procedure - current Maintenance procedures do not specifically require the completion of local leak rate tests following maintenance on containment isolation valves. Communications, both verbal and the written Maintenance Order, did not identify the valve diaphragm replacement would result in a potential loss of containment integrity.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 9/31/85

FACILITY NAME (1) BIG ROCK POINT PLANT	DOCKET NUMBER (2) 0500015585	LER NUMBER (3)			PAGE (3)					
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER						

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Immediate Corrective Action Taken

1. Containment integrity was re-established by closing CV-4025 as discussed above.
2. A review of all planned, outstanding and completed maintenance activities on containment isolation valves was performed. This concluded that no other valves were affected by this type of occurrence.

Proposed Corrective Action to Prevent Recurrence

1. A review of Operations Department controls pertaining to containment integrity and authorization of maintenance activities will be performed and modified as necessary to identify the need to perform component leak rate tests following maintenance on containment boundary components.
2. A review of existing maintenance controls and procedures pertaining to containment isolation valves and modify as necessary to identify the need to perform component leak rate testing following the maintenance activity prior to declaring the equipment operable.

The above actions will be completed by April 1, 1986.



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DOCKET 50-155 - LICENSE DPR-6 - BIG ROCK POINT PLANT -
LICENSEE EVENT REPORT 85-007 - INADEQUATE CONTROLS AS
DESCRIBED BY TECHNICAL SPECIFICATIONS

Licensee Event Report (LER) 85-007 (Inadequate Controls as Described by
Technical Specifications) is attached. This event is reportable to the NRC
per 10CFR50.73(a)(2)(vii).

Brian D Johnson
Staff Licensing Engineer

CC Administrator, Region III, USNRC
NRC Resident Inspector - Big Rock Point

Attachment

IE22
11