

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) RANCHO SECO NUCLEAR GENERATING STATION UNIT NO. 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 1 2	PAGE (3) 1 OF 0 1 2
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TITLE (4)

NSCW PUMP "A" BREAKER

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 7	0 9	8 5	8 5	0 1 4	0 0	0 8	0 2	8 5	NONE		0 5 0 0 0

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)											
	20.402(b)			20.405(c)			50.73(a)(2)(iv)			73.71(b)		
	20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)			73.71(c)		
	20.405(a)(1)(ii)			X 50.36(c)(2)			50.73(a)(2)(vi)			OTHER (Specify in Abstract below and in Text, NRC Form 366A)		
	20.405(a)(1)(iii)			50.73(a)(2)(i)			50.73(a)(2)(viii)(A)					
POWER LEVEL (10) 0 1 0	20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)					
	20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(ix)					

LICENSEE CONTACT FOR THIS LER (12)

NAME Ron W. Colombo, Regulatory Compliance Supervisor	TELEPHONE NUMBER	
	AREA CODE 9 1 6	4 5 2 - 3 2 1 1 L

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPDOS

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			0 8	3 1 0	8 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 14, 1985, at 1522 hours, the reactor was taken critical following a refueling outage and reached 14% thermal power at 1131 hours on June 18, 1985, before initiating shutdown to investigate a clogged turbine lube oil system line. During this period of operation the Nuclear Service Cooling Water (NSCW) "A" pump breaker was a Class 2 breaker which, unknown to operations personnel, lacked the necessary completed documentation to be utilized in a Class 1 application. Operation of the plant during this period resulted in a violation of Technical Specification 3.3.1. This event report is being submitted in accordance with 10 CFR 50.36(c)(2).

The discrepancy was detected on July 9, 1985, while an engineer was reviewing the associated breaker documentation. The NRC was notified of the breaker concern via the Control Room "red" phone within four (4) hours of its detection. Because the plant was in a cold shutdown condition no immediate corrective action was required.

A preliminary investigation has determined the cause of this event to be a breakdown in programmatic controls. The District will perform a root cause analysis of this event and the results of this analysis will be submitted in a supplement to this report by August 30, 1985.

There were no effects on plant or public safety as a result of this event.

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/95

FACILITY NAME (1)  RANCHO SECO NUCLEAR GENERATING STATION UNIT NO. 1	DOCKET NUMBER (2)  050003112	LER NUMBER (6)			PAGE (3)		
		YEAR 85	SEQUENTIAL NUMBER 0114	REVISION NUMBER 010			

TEXT (If more space is required, use additional NRC Form 366A (2) (17))

On June 14, 1985, at 1522 hours, the reactor was taken critical following a refueling outage and reached 14% thermal power at 1131 hours on June 18, 1985, before initiating shutdown to investigate a clogged turbine lube oil system line. During this period of operation the Nuclear Service Cooling Water (NSCW) "A" pump breaker was a Class 2 breaker installed on June 13, 1985 which, unknown to operations personnel, lacked the necessary documentation to be utilized in a Class 1 application. The NSCW pump "A" breaker was identical to the Class 1 breaker it had replaced on June 13, 1985, but had not been procured and maintained as a Class 1 device. Operation of the plant during this period resulted in a violation of Technical Specification 3.3.1 which does not allow the reactor to remain critical unless two (2) NSCW pumps are operable or maintenance is being performed to repair the inoperable pump within 48 hours. This event report is being submitted in accordance with 10 CFR 50.35(c)(2).

An additional attempt to achieve power operation was made on June 23, 1985. This attempt was aborted because of mechanical problems discovered prior to reaching reactor criticality.

The Class 2 breaker was installed on June 13, 1985 to replace a failed breaker. The replacement breaker was satisfactorily tested prior to being racked into the NSCW "A" train.

On July 8, 1985 during a routine surveillance test, the breaker failed to close on command. As part of the investigation of this failure, the lack of documentation for justifying the NSCW "A" pump breaker in a Class 1 application was detected on July 9, 1985, while an engineer was reviewing the associated breaker documentation. The NRC was notified of the breaker discrepancy via the Control Room "red" phone at 1641 hours on July 9, 1985 within four (4) hours of its detection. Because the plant was in a cold shutdown condition with the decay heat system in service being cooled by the "B" NSCW train, no immediate corrective action was necessary. A Class 1 replacement breaker for the "A" NSCW train was not available, so interim instructions were given to Control Room personnel on July 12, 1985 prohibiting the use of NSCW train "A" except in the case of an emergency. A Class 1 breaker was repaired and placed in service on July 24, 1985.

The District has formed a task force whose overall objective is to determine the root cause of the NSCW breaker's failure to close on command during the July 8, 1985 test, and the lack of adequate documentation on the installed breaker. A preliminary investigation by the team has determined the cause of the incomplete Class 1 documentation event to be a breakdown in programmatic controls governing the spare parts replacement of Class 1 equipment. A sub-task of this group is to examine existing procedural controls and determine the necessary changes to prevent recurrence of the event. The results of this team investigation and their recommended corrective action will be submitted by August 30, 1985 in a supplement to this report.

There were no effects on plant or public safety as a result of this event.



**SMUD**

SACRAMENTO MUNICIPAL UTILITY DISTRICT ☐ 6201 S Street, P.O. Box 15830, Sacramento CA 95852-1830, (916) 452-3211  
AN ELECTRIC SYSTEM SERVING THE HEART OF CALIFORNIA

RJR 85-369

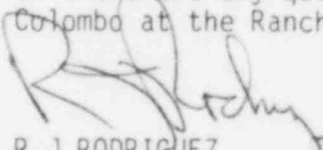
August 2, 1985

J B MARTIN REGIONAL ADMINISTRATOR  
REGION V OFFICE OF INSPECTION AND ENFORCEMENT  
ATTN DOCUMENT CONTROL DESK  
U S NUCLEAR REGULATORY COMMISSION  
WASHINGTON DC 20555

DOCKET NO. 50-312  
LICENSE NO. DPR-54  
LICENSEE EVENT REPORT NUMBER 85-14

In accordance with the requirements of 10 CFR 50.36(c)(2), the Sacramento Municipal Utility District hereby submits Licensee Event Report Number 85-14.

If there are any questions concerning this report, please contact Mr. Ron Colombo at the Rancho Seco Nuclear Generating Station Unit No. 1.

  
R J RODRIGUEZ  
ASSISTANT GENERAL MANAGER,  
NUCLEAR

Attachment

cc: Region V (2)  
INPO

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