

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Fort St. Vrain, Unit No. 1DOCKET NUMBER (2)  
0 5 0 0 0 1 2 6 7 1 OF 0 1 3TITLE (4)  
Unplanned Actuation Of Scram Circuitry During Investigative Testing

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)		
0	9	2	5	8	5	0	1	8	N/A	0 5 0 0 0 1 2 6 7 1		
0	9	2	5	8	5	0	0	1		0 5 0 0 0 1 2 6 7 1		

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)

OPERATING MODE (9)	POWER LEVEL (10)	20.402(b)	20.405(a)(1)(i)	20.405(a)(1)(ii)	20.405(a)(1)(iii)	20.405(a)(1)(iv)	20.405(a)(1)(v)	20.405(c)	20.405(c)(1)	20.405(c)(2)	20.405(c)(2)(i)	20.405(c)(2)(ii)	20.405(c)(2)(iii)	20.405(c)(2)(iv)	20.405(c)(2)(v)	73.71(b)	73.71(c)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
N	0.00																	

LICENSEE CONTACT FOR THIS LER (12)  
NAME  
Jim Eggebroten, Superintendent, Technical Services Eng.TELEPHONE NUMBER  
AREA CODE  
3 0 3 7 8 5 1 - 2 2 2 4COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)  

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
A	J	O	F	U					

SUPPLEMENTAL REPORT EXPECTED (14)  
☐ YES (If yes, complete EXPECTED SUBMISSION DATE)  
☒ NOEXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 25, 1985, with the reactor shutdown for primary coolant clean up, an automatic actuation of the Plant Protective System (PPS) reactor scram circuitry occurred. The PPS reactor scram occurred as a result of a personnel error during troubleshooting on "D" Circulator inlet temperature.

Personnel error caused a short and subsequent loss of power to the 100 Bay of the PPS. As a result of the loss of power, all "B" channel scram inputs were actuated.

The technician was advised that in the future he should seek alternative methods or request technical assistance when attaching leads for testing in congested areas. When alternative methods are not available in congested areas, extreme caution should be used in attaching leads for testing.

The automatic actuation of the PPS reactor scram circuitry is being reported per 10CFR50.73(a)(2)(iv).

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)  Fort St. Vrain, Unit No. 1	DOCKET NUMBER (2)  0 5 0 0 0 2 6 7 8 5 - 0 1 8 - 0 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 368A's) (17)

EVENT DESCRIPTION:

On September 25, 1985, at approximately 1306 hours, a PPS reactor scram actuation occurred on High Startup Count Rate.

In the process of troubleshooting "D" Circulator helium inlet temperature indication anomalies in the PPS, an instrument technician shorted a circuit. The short caused a loss of power to the 100 Bay of the PPS. As a result of the loss of power, all "B" channel scram inputs were actuated. Because the Reactor Mode Switch was in the "Off" position, the High Startup Count Rate scram of the startup channels was not bypassed. One of the two channels for this one of two scram logic is located in the 100 Bay of the PPS. The loss of power resulted in a trip signal for this channel and therefore a reactor scram actuation.

ANALYSIS OF EVENT:

The reactor had been shutdown since July 24, 1985, for primary coolant cleanup. All thirty-seven control rod pairs were fully inserted.

The automatic actuation of the plant protective system circuitry did not result from a change in core reactivity. The plant protective system action was conservative and functioned as designed.

Had the reactor mode switch been in the "Fuel Loading" position, the loss of power in the 100 Bay of the PPS would have resulted in the same action.

Had the reactor mode switch been in the "Run" position, no automatic actuation of the plant protective system circuitry would have occurred. The loss of power in the 100 Bay of the PPS would have resulted in actuating one channel in the two out of three logic necessary to initiate a PPS reactor scram.

CAUSE DESCRIPTION:

The cause of the event is attributed to personnel error while troubleshooting "D" Circulator helium inlet temperature indication anomalies in the PPS.

CORRECTIVE ACTION:

The technician was advised that in the future he should seek alternative methods or request technical assistance when attaching leads for testing in congested areas. When alternative methods are not available, extreme caution should be used when attaching leads for testing in congested areas.

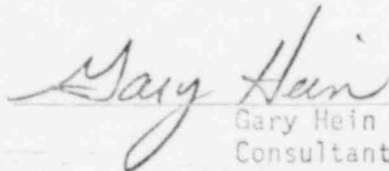
All instrument technicians were given the same caution.

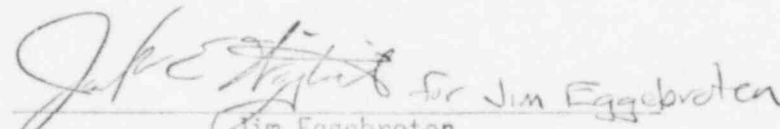
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EXPIRES 8/31/85

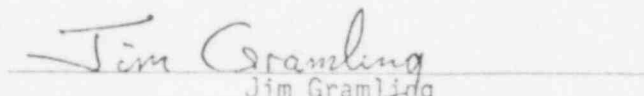
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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

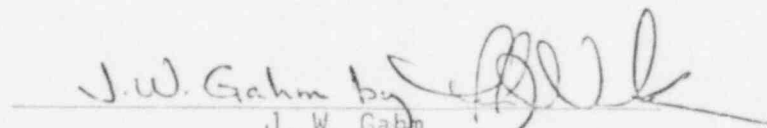
  
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Licensing Review By: 

  
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Company of Colorado

October 16, 1985  
Fort St. Vrain  
Unit No. 1  
P-85379

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Docket No. 50-267

SUBJECT: Licensee Event Report  
85-018, Final Report

REFERENCE: Facility Operating  
License No. DPR-34

Gentlemen:

Enclosed please find a copy of Licensee Event Report  
No. 50-267/85-018, Final, submitted per the requirements of  
10 CFR 50.73(a)(2)(iv).

Sincerely,

*J.W. Gahm by [Signature]*  
J. W. Gahm  
Manager, Nuclear Production

Enclosure

cc: Regional Administrator, Region IV  
Attn.: Mr. E. H. Johnson  
Chief Reactor Projects Branch

cc: Director of Nuclear Reactor Regulation  
Attn.: Mr. E. J. Butcher, Jr.  
Acting Chief Operating Reactors Branch No. 3

cc: Director, MIPC

JWG/djm

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