

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Millstone Nuclear Power Station Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 2 4 5										PAGE (3) 1 OF 0 2																																			
TITLE (4) Reactor Scram																																																							
EVENT DATE (5)									LER NUMBER (6)									REPORT DATE (7)									OTHER FACILITIES INVOLVED (8)																												
MONTH			DAY			YEAR			YEAR			SEQUENT AL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES													DOCKET NUMBER(S)															
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OPERATING MODE (9)									THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following): (11)																																														
POWER LEVEL (10) 0 1 5									20.402(b)									20.406(c)									<input checked="" type="checkbox"/> 50.73(a)(2)(iv)									73.71(b)																			
									20.406(a)(1)(B)									50.36(c)(1)									50.73(a)(2)(v)									73.71(c)																			
									20.406(a)(1)(K)									50.36(c)(2)									50.73(a)(2)(vi)									OTHER: Specify in Abstract below and in Text, NRC Form 366A.																			
									20.406(a)(1)(H)									50.73(a)(2)(ii)									50.73(a)(2)(vii)(A)																												
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20.406(a)(1)(V)									50.73(a)(2)(H)									50.73(a)(2)(k)																																					
LICENSEE CONTACT FOR THIS LER (12)																																																							
NAME David Yapchanyk, Engineer X 4428																				TELEPHONE NUMBER AREA CODE 2 0 3 4 4 7 - 1 7 9 1																																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																							
CAUSE		SYSTEM		COMPONENT		MANUFAC TURE		REPORTABLE TO NPROS		CAUSE		SYSTEM		COMPONENT		MANUFAC TURE		REPORTABLE TO NPROS																																					
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SUPPLEMENTAL REPORT EXPECTED (14)																																																							
YES (if yes, complete EXPECTED SUBMISSION DATE):																				<input checked="" type="checkbox"/> NO										EXPECTED SUBMISSION DATE (15)																									
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ABSTRACT (Limit to 1400 spaces. i.e., approximately fifteen single space typewritten lines) (16)

On September 27, 1985 at 1210 hours while conducting a reactor shutdown the reactor scrambled on average power range monitor (APRM) Hi Hi when the reactor mode switch was changed from "Run" to "Startup/Hot Stby." Reactor power was at approximately 15% prior to the scram. Off normal procedures were followed by operations personnel. All of the safety systems functioned as required and the reactor was brought to hot shutdown.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Millstone Nuclear Power Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 4 5	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
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TEXT (If more space is required, use additional NRC Form 365A's) (17)

On September 27, 1985 at 1210 hours while conducting a reactor shutdown the reactor scrammed on average power range monitor (APRM) Hi Hi when the reactor mode switch was changed from "Run" to "Startup/Hot Stby." Reactor power was approximately 15% prior to the scram. Off normal procedures were followed by operations personnel. All of the safety systems functioned as required and the reactor was brought to hot shutdown.

The reactor operator was following OP 205, Planned Shutdown to Hot Standby or Hot Shutdown, in anticipation of hurricane Gloria. All of the intermediate range monitor (IRM) detectors were inserted, as directed by the procedure. A caution statement in the procedure directs the operator to "ensure that no IRM Hi Hi or INOP trip indicators are lighted prior to placing the reactor mode switch in start," or a reactor scram will result. None of the IRM trip indicators were lit. The procedure then directs the operator to place the reactor mode switch in the "start up" position prior to reaching an APRM level of 5%.

Knowing that a reactor scram would occur on APRM Hi Hi if the mode switch was taken out of "Run" with reactor power at 15% or greater, the reactor operator checked the IRM/APRM recorders on CRP 905, which were reading approximately 13%. The reactor operator then began moving the mode switch to "Startup," resulting in a reactor scram on APRM Hi Hi.

Instrument and Control Surveillance Procedure SP 404C, APRM Calibration, requires that the APRM setdown trip Hi Hi setpoint be $14\% \pm 1$. The Technical Specification limit is 15%. SP 404B, APRM Functional Test, was performed on 10/7/85 and showed that five APRM channels Hi Hi trips were set at 14% power, and the other channel was set at 13.5%. Thus, the mode switch was moved to "Startup/ Hot Stby" with indicated power below 15%, but not below the APRM Hi Hi trip setting which caused a reactor scram. As a result of the reactor scram, off normal procedures were followed by operations personnel and the reactor was brought to hot shutdown.

The incident was discussed with all of the operators and OP 205, Planned Shutdown to Hot Standby or Hot Shutdown, was revised to ensure that reactor power has been decreased to 5% to avoid the APRM Hi Hi scram when the mode switch is moved from "Run" to "Start up/Hot Stby."

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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October 25, 1985

MP-8312

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Reference: Facility Operating License No. DPR-21
Docket No. 50-245
Reportable Occurrence RO-85-017-00

Gentlemen:

This letter forwards the Licensee Event Report 85-017-00 required to be submitted within thirty days, pursuant to the requirements of 10CFR50.73.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

A handwritten signature in cursive script, appearing to read 'Wayne D. Romberg'.

Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

WDR/DY:ejl

Attachment: LER 50-245/85-017-00

cc: Dr. T. E. Murley, Region I

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