NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Zion, Unit 2	0 5 0 0 0 3 0 4	1 OF 0 2

TITLE (4)
Automatic Closure of Blowdown Containment Isolation Valves

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 7	0 7	8 5	8 5	0 1 3	0 0	0 8	0 6	8 5		0 5 0 0 0 2 9 5
										0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 7 9	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Greg Kassner	3 1 2 — 7 4 6 - 2 0 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	W I			N							
A	I L			N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

During normal operation an attempt was made to restore Unit 2 steam generator blowdown, which had been isolated for approximately 21 hours. Shortly after restoring flow the blowdown radiation monitor high alarmed closing the blowdown isolation valve BD-17, which is an engineered safety feature. Contrary to Abnormal Operating Procedure, AOP-5 the radiation/chemistry department was not notified and a second attempt was made to establish blowdown flow with the same result. The increased blowdown activity was probably due to a known primary-secondary tube leak of approximately 25 gpd combined with the 21 hour period of blowdown isolation.

The Operating Department will be reminded of the requirement to contact the Radiation/Chemistry department of radiation monitor high alarm conditions.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
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Zion, Unit 2	0 5 0 0 0 3 0 4	8 5	— 0 1 3	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During normal operation of Units 1 and 2 on 7/7/85, at approximately 0725 hours, an attempt was made to restore U-2 blowdown. U-2 blowdown had been isolated for approximately 21 hours prior to the attempt in order to facilitate utilization of both demin trains for U-1 blowdown. Shortly after U-2 blowdown was restored, the steam generator blowdown radiation monitor, 2RE0019, high alarmed causing isolation of U-2 blowdown and its associated sample system, which is an actuation of an engineered safety feature. The blowdown isolation valve, BD-17 was verified closed, an NRC red phone call was initiated per 10CFR50.72, and the load dispatcher and operating engineer were notified. No safety implications resulted from the incident.

Additionally, after the high alarm, the monitor was reset and a second attempt was made to establish blowdown. The second attempt also resulted in a high alarm on 2RE0019. Contrary to the requirements of the Abnormal Operating Procedure, AOP-5, section 6.0.A.6.2.1, shift supervision did not notify the Radiation/Chemistry department of the high alarm. Consequently, the High Radiation Monitor Alarm procedure, RP 1350-37, was not performed by Radiation/Chemistry and thus, no confirmatory grab sample was obtained from the blowdown sample system.

In a follow up discussion with shift supervision, the Shift Control Room Engineer explained that pressure control valve OPCV-BD04 was stroking full open instantaneously, likely causing abnormal flow conditions at the monitor which could have yielded the monitor high alarms. At approximately 0830 hours on 7/7/85, a third attempt to start blowdown was made and OPCV-BD04 was opened manually. This attempt did not result in a high alarm and blowdown was maintained.

The chart recorder trace for 2RE0019, later inspected by Radiation/Chemistry indicated that elevated activity was present at the monitor. The elevated readings, approximately 3 times greater than recent background readings, also could have resulted from a simultaneous existence of two conditions. 1) 2RE0019 may have been at the "C" steam generator sampling stage of the monitor sample cycle at the time blowdown was being restored. (A primary to secondary leak of approximately 25 gpd has existed in "C" steam generator since 6/4/85 and routinely causes elevated but "non-alarm condition" readings on 2RE0019). 2) Steam generator blowdown activity may have accumulated during the 21 hour period in which no demineralization was in effect.

An INPO Computer LER Data Base search was performed to identify previous occurrences of this type but no such incidents have been reported to date. U-2 blowdown has not been isolated due to a 2RE0019 monitor high alarm since the event on 7/7/85.

The Radiation/Chemistry department will issue a notice to the Operating department which re-iterates that Operating must notify Radiation/Chemistry of every radiation monitor high alarm so that the alarm can be verified with appropriate samples. No additional corrective action is necessary.

Commonwealth Edison
Zion Generating Station
101 Shiloh Blvd.
Zion, Illinois 60099
Telephone 312/746-2084

August 6, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

References: 10CFR50

Dear Sir:

The enclosed Licensee Event Report from Zion Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73 (a)(2)(iv) which requires a 30 day written report when an event or condition results in manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS).

This report is number 85-013-00, Docket No. 50-304/DPR-48.

Very truly yours,

K. L. Graesser
Station Manager
Zion Generating Station

KLG/rmm

Enclosure: Licensee Event Report No. 85-013-00

Attachment

cc: J. G. Keppler, NRC Region III Administrator
M. Holzmer, NRC Resident Inspector
INPO Record Center
CECo Distribution List

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