NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Fermi-2	0 5 0 0 0 3 4 1	1 OF 0 3

TITLE (4)

Reactor Trip - Returning Instrument to Service

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 9	2 8	8 5	8 5	0 6 7	0 0	1 0	2 5	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
L.P. Bregni, Compliance Engineer	AREA CODE 3 1 3 5 8 6 - 5 3 1 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 28, 1985 at 0657 hours, a full scram signal was generated while restoring a reactor vessel instrument (Jet Pump Developed Head transmitter) to service following calibration. The plant was in Operational Condition 3 (Hot Shutdown) with control rods inserted and reactor pressure about 45 psig. The trip was caused by a spurious low reactor vessel water level 3 signal resulting from a pressure transient induced by incorrect valving technique. The transmitter's calibration package did not identify the instrument as being on a common instrument sensing line with several other reactor instruments, which resulted in an incorrect section of the procedure being used to return the transmitter to service. Corrective action includes the addition of all common leg instruments to the valving procedure to prevent confusion (procedure change submitted October 9, 1985), addition of a cautionary statement on all common line instrument's Specification Sheets which are part of the calibration package.

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On September 28, 1985 at 0657 hours, with the plant in Operational Condition 3 (Hot Shutdown) and reactor pressure about 45 psig, a full scram signal was generated as the result of an erroneous reactor vessel level 3 indication. The scram signal occurred while the "Jet Pump Developed Head" transmitter (B21N035) was being returned to service after calibration. All control rods, SRMs and IRMs were inserted prior to the scram signal. Valve Groups 4 and 15 were closed before the scram, and Group 13 valves isolated.

The scram signal was the result of Instrument and Control (I&C) personnel improperly valving transmitter B21N035 (Jet Pump Developed Head) into service without filling and pressurizing the instrument and associated instrument sensing lines. The resulting pressure spike created a perturbation in the reference line (of the reactor level instruments which share the line) of sufficient magnitude to exceed the level 3 setpoint.

The improper valving action resulted from I&C personnel failing to recognize that the sensing line to the low input of B21N035 was connected to a sensing line common to the reference line of seven (7) reactor vessel level instruments. The calibration package did not identify B21N035 as an instrument utilizing a common sensing line. This caused the I&C personnel to use an incorrect section of procedure 41.000.09, "Process Instrumentation Removal From and Return to Service" when they were returning the instrument to service. Had it been properly identified in the calibration package, the section in the procedure they would have used provides a caution to alert personnel to the potential for causing a reactor scram when returning the instrument to service.

Corrective actions have been taken as follows:

1. I&C personnel have been counseled concerning the "attention to detail" required when working on or around RPS or ECCS instrumentation.

2. RPS and ECCS instruments utilizing common sensing lines have been listed in Procedure 41.000.09.

3. Specification Sheets for RPS and ECCS instrumentation utilizing common sensing lines have been revised to include "Caution, Shared Sensing Line", to alert I&C personnel to the high probability of a scram, isolation, actuation, etc. if valved incorrectly. The Specification Sheets are included in the calibration package.

Similar events were disscussed in LER's 85-014, -015, -016, and -021, in which RPS trips resulted from personnel valving level instruments back into service. In LER 85-021, corrective action taken included revising procedure 41.000.09, "Process Instrumentation Removal From and Return to Service". The revision clarified that the precautions on returning reactor vessel instruments to service apply to all instruments on racks H21P004 and H21P005, and to instrument B21N086A on rack H21P009 and B21N086B on rack H21P010. However, transmitter B21N035 on instrument rack H21P010 was inadvertantly overlooked when 41.000.09 was revised and was not included in the precaution. As noted above, the procedure has been revised to include the transmitter and a review has been conducted to ensure no others were left out of this revision.

Robert S. Lenart
Plant Manager

Detroit Edison

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October 25, 1985
NP850174

Nuclear Operations

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Reference: Fermi 2
NRC Docket No. 50-341
NRC Operating License No. NPF-43

Subject: Transmittal of Licensee Event Report 85-067

Please find enclosed LER No. 85-067-00, dated October 25, 1985, for a reportable event which occurred on September 26, 1985. As indicated below, a copy of this LER is being sent to the Administrator Region III.

If you have any questions, please contact us.

Sincerely,

R. S. Lenart
Plant Manager

Enclosure: NRC Forms 366, 366A

cc: P.M. Byron
M.D. Lynch

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