



UNITED STATES
NUCLEAR REGULATORY COMMISSION

REGION IV

611 RYAN PLAZA DRIVE, SUITE 400
ARLINGTON, TEXAS 76011-8064

JAN 21 1996

Harold B. Ray, Executive Vice President
Southern California Edison Co.
San Onofre Nuclear Generating Station
P.O. Box 128
San Clemente, California 92674-0128

SUBJECT: NRC INSPECTION REPORT 50-361; -362/96-13

Dear Mr. Ray:

An NRC inspection was conducted September 16-20, December 9-13, 1996, and on January 14, 1997, at your San Onofre Nuclear Generating Station, Units 2 and 3 reactor facilities. This inspection evaluated the self-assessment of your engineering and fire protection activities which your staff performed as an alternative to a planned NRC team inspection. The enclosed report presents the scope and results of the inspection. The final results of this inspection were discussed with Mr. G. Gibson of your staff on January 9 and 14, 1997.

Our inspection found that your self-assessment of engineering was a thorough and comprehensive alternative to the planned NRC team inspection in this area. Your self-assessment concluded that your engineering activities met your program requirements. Our independent inspection generally confirmed that conclusion. Therefore, in accordance with NRC Inspection Procedure 40501, "Licensee Self-Assessments Related to Team Inspections," the NRC plans no further inspection effort during the current performance assessment cycle at the San Onofre Nuclear Generating Station for NRC Inspection Procedure 37550, "Engineering." Although your self-assessment of fire protection concluded that your activities were effective in preventing, detecting, and responding to a fire emergency, it did not adequately address several important inspection activities. Therefore, an additional NRC inspection for NRC Inspection Procedure 64704, "Fire Protection Program," will be performed during the current performance assessment cycle to confirm your self-assessment conclusions in this area.

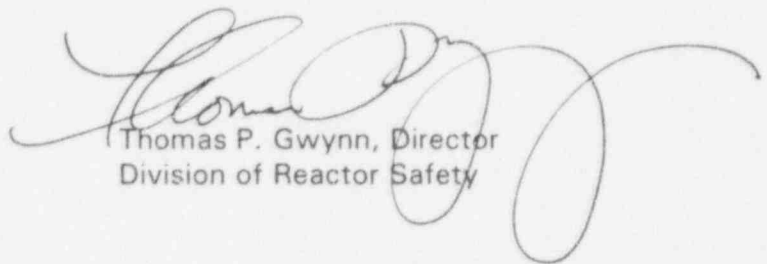
One unresolved item was identified concerning the adequacy of your 10 CFR 50.59 evaluation for your diesel generator cross-tie modification. This matter is discussed in Section E1.5.b(2) of the enclosed inspection report. Specifically, your self-assessment questioned the adequacy of the 10 CFR 50.59 evaluation that had been performed for a design change involving the installation of electrical components that would give operators the ability to promptly cross-connect an emergency diesel generator from one unit to the safety bus of another unit during an emergency 10 CFR 50.54(x) condition. The

inspectors questioned whether the physical installation of the design change in Unit 2 could result in an unreviewed safety question during normal operation of the unit. In response to this concern, we understand that you have electrically isolated the modification and submitted a description of the modification to the NRC for further evaluation.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, its enclosure(s), and your response will be placed in the NRC Public Document Room (PDR).

Should you have any questions concerning this inspection, we will be pleased to discuss them with you.

Sincerely,



Thomas P. Gwynn, Director
Division of Reactor Safety

Docket Nos. 50-361; 50-362
License Nos. NPF-10; NPF-15

Enclosure:
NRC Inspection Report
50-361/96-13; 50-362/96-13

cc w/enclosure:
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E-Mail report to D. Nelson (DJN)

E-Mail report to NRR Event Tracking System (IPAS)

bcc to DMB (!E01)

bcc distrib. by RIV:

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 Branch Chief (DRP/F, WCFO)
 Senior Project Inspector (DRP/F, WCFO)
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E-Mail report to D. Nelson (DJN)
E-Mail report to NRR Event Tracking System (IPAS)

bcc to DMB (IE01)

bcc distrib. by RIV:

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Branch Chief (DRP/TSS)
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