

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3180-010W
EXPIRES - 9/31/95

FACILITY NAME (1)										INDIAN POINT UNIT 2										DOCKET NUMBER (2)										PAGE (3)									
TITLE (4)										REACTOR TRIP: STEAM GENERATOR SWELL										0 5 0 0 0 2 4 7										1 OF 0 3									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																													
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME						DOCKET NUMBER (9)																								
0	9	2	3	8	5	0	1	0	0	0	1	0	2	3	8	5	0 5 0 0 0																						
OPERATING MODE (10)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																				
POWER LEVEL (10)			20.402(a)				20.403(a)				X 80.73(a)(2)(iv)				73.71(a)																								
0 1 2			20.406(a)(1)(i)				80.36(a)(1)				80.73(a)(2)(v)				73.71(a)																								
			20.406(a)(1)(ii)				80.36(a)(2)				80.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 266A)																								
			20.406(a)(1)(iii)				80.73(a)(2)(i)				80.73(a)(2)(iii)(A)																												
			20.406(a)(1)(iv)				80.73(a)(2)(ii)				80.73(a)(2)(iv)(B)																												
			20.406(a)(1)(v)				80.73(a)(2)(iii)				80.73(a)(2)(v)																												
LICENSEE CONTACT FOR THIS LER (12)																																							
NAME										TELEPHONE NUMBER																													
JOHN R. ELLWANGER										AREA CODE		9 1 1 4					5 2 1 6 1 5 1 8 2																						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																							
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC																				
SUPPLEMENTAL REPORT EXPECTED (14)													EXPECTED SUBMISSION DATE (15)				MONTH DAY YEAR																						
YES (If yes, complete EXPECTED SUBMISSION DATE)													X NO																										

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

The reactor tripped on September 23, 1985 during restart from a Unit trip reported in LER 85-009-00. A High Steam Generator level signal resulted in a main generator trip which, in turn, caused a turbine trip/reactor trip. The cause of the trip is attributed to operator failure to anticipate a steam generator swell at the time of synchronization of the generator output to the bus and pick up of load.

The reactor protection system functioned normally and there were no abnormalities observed in other safety related equipment.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (5)

PAGE (3)

INDIAN POINT UNIT 2

YEAR

SEQUENTIAL
NUMBERREVISION
NUMBER

0 5 0 0 0 2 4 7 8 5 - 0 1 0 - 0 0 0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 345a) (17)

Plant and System Identification:

Westinghouse 4-loop pressurized water reactor; 900 MWe.

Identification of Occurrence:

Reactor Trip High Steam Generator Level.

Event Date: 9/23/85Reportability Determination Date: 9/26/85Report Due Date: 10/20/85

This report was initiated by SOR 85-423.

Description of Occurrence:

On September 23, 1985 at 9:55 p.m., during restart of the plant, a reactor trip occurred. The reactor was at 12% power and the main generator breakers had just been closed when a high steam generator level signal from #22 steam generator resulted in a turbine generator trip immediately followed by a reactor trip. The Reactor Protection System and other safety related equipment functioned normally.

Apparent Cause of Occurrence:

An evaluation of level indication from the affected steam generator showed that all three channels read less than 72% full just prior to the main generator trip. The trip setpoint corresponds to 75%. A surge within the steam generator, due to load pick up immediately following the time the main output breakers closed, caused the trip and there was no equipment malfunction. The operators failed to anticipate this occurrence.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 3/31/95

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (5)

PAGE (3)

INDIAN POINT UNIT 2

YEAR SEQUENTIAL REVISION

NUMBER NUMBER NUMBER

0 5 0 0 0 2 4 7 8 5 - 0 1 0 - 0 0 0 3 OF 0 3

TEXT (If more space is required, use additional NRC Form 266A (17))

Analysis of Occurrence:

The event represented an unplanned actuation of the Reactor Protection System. All equipment in this respect functioned normally. There was no impact on safety.

Corrective Action:

The event and the appropriate measures to be implemented to avoid a reactor trip were reviewed with the Operations staff.

John D. O'Toole
Vice President

Consolidated Edison Company of New York, Inc.
4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

October 23, 1985


Re: Indian Point Unit No. 2
Docket No. 50-247
LER-85-010-00

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sirs:

The attached Licensee Event Report LER-85-010-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,


for J. D. O'Toole

attach.

cc: Dr. Thomas E. Murley,
Regional Administrator - Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

Senior Resident Inspector
U. S. Nuclear Regulatory Commission
P. O. Box 38
Buchanan, New York 10511

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