



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

CONSUMERS POWER COMPANY

DOCKET NO. 50-155

BIG ROCK POINT PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79
License No. DPR-6

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consumers Power Company (the licensee) dated June 27, 1985 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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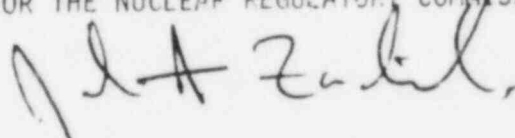
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraphs 2.C.(2) of Facility Operating License No. DPR-6 are hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 79, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "John A. Zwolinski", is written over the typed name below.

John A. Zwolinski, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 22, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 79

FACILITY OPERATING LICENSE NO. DPR-6

DOCKET NO. 50-155

Revise Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

5-16a
6-7
6-7a
6-7b
6-9
7-9

INSERT

5-16a
6-7
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6-9
7-9

.3 Liquid Poison System

The liquid poison system shall be available for operation at all time during refueling and power operation. The reactor shall be shutdown in any situation where the poison solution tank level drops below an equivalent of 850 gallons 19 weight percent sodium pentaborate or where the poison solution storage temperature drops to less than 5°F above saturation temperature. The maximum allowable concentration shall be 30 weight percent of sodium pentaborate. The minimum worth of the liquid poison system (based on normal water level) shall be 25% Δk_{eff} . Components of the system shall be checked at one to two month intervals for proper operation except for actuation of the injection valves. The liquid poison system shall be used at any time when subcriticality cannot be assured by the normal shutdown mechanism. Injection shall be continued until a minimum shutdown margin of $0.01\Delta k_{eff}/k_{eff}$ is assured in the most reactive core. The reactor shall not be operated after poison has been injected until the boron concentration in the reactor water has been reduced to 100 ppm or less. One squib primer and trigger assembly from the equalizing line shall be removed and test-fired at least every 18 months. These shall be tested on an alternate basis ensuring valve replacement every 36 months. One squib primer and trigger assembly from the remaining five units shall be removed and test-fired at least every 18 months. These shall be tested on an alternate basis. In no case shall a squib primer and trigger assembly remain in service longer than five years. The tests shall consist of monitoring of the input firing current and shearing of the integral inlet cap.

6.1.3 (Contd.)

- (c) With the mode switch in the "shutdown" position, both the scram circuit and the control rod withdrawal circuit are open. The ventilating duct circuit power supply is transferred to a point which provides penetration closure protection through signals from "high containment sphere pressure" and "low water level in reactor vessel." This permits normal ventilation in the containment sphere during shutdown when the control rods are held in the full-in position. None of the reactor safety system signals are bypassed since there is no need to withdraw control rods.
- (d) With the mode switch in the refuel position and the crane positioned over the reactor vessel, crane operation is prevented if any one rod is withdrawn from full-in position.
- (e) High condenser pressure reactor trip is automatically bypassed any time steam drum pressure is below a set point maximum of 500 psig.

6.1.4 Related Systems

(a) Emergency Condenser Control

A pressure switch shall initiate automatic operation of the emergency condenser if the reactor pressure reaches 100 ± 10 psi above the reactor operating pressure.

6.1.5 Operating Requirements

- (a) Except as otherwise provided in these Technical Specifications, the reactor safety system shall be operable during power operation as indicated in Section 6.1.2. This system shall be functionally tested during each major refueling shutdown, but not less frequently than once every 18 months and in addition shall be tested not less frequently than once a month using the switches provided to simulate sensor trips.
- (b) The emergency condenser system control initiation sensors shall be functionally tested at each major refueling shutdown but not less frequently than once every 18 months.

6.2 CONTROL ROD WITHDRAWAL PERMISSIVE SYSTEM

6.2.1 Interlocks

Interlocks shall prevent control rod withdrawal when any of the following conditions exist:

- (a) When any two of the thirty-two scram accumulators are at pressure below 700 psig.
- (b) When two of the three power range channels read below 5 percent on their 0 to 125 percent scales (or below 2 percent on their 0-40 percent scales) when reactor power is above the minimum operating range of these channels. This interlock may be bypassed when all three of the power range channels are set on the minimum operable range.
- (c) When the scram dump tank is bypassed.
- (d) When the mode selector switch is in the shutdown position.

6.2.2 Operating Requirements

The control rod withdrawal permissive interlocks shall always be operable. No further withdrawal of control rods will be permitted if one of these circuits is found to be inoperable.

Permissive circuits shall be functionally tested prior to each major refueling but no less frequently than every 18 months. However, the refueling interlocks will be functionally tested prior to each major refueling.

6.3 REFUELING OPERATION INTERLOCK SYSTEM

6.3.1 Reactor Refueling System

All of the trip devices not bypassed by the mode selector switch in the refuel position shall be operative during all refueling operations. This shall include the sensors and trip devices of the reactor safety system as specified for power operation as follows:

- High Reactor Pressure
- Low Reactor Water Level
- High Containment Sphere Pressure
- High Scram Dump Tank Level
- Loss of Auxiliary Power Supply
- High Neutron Flux
- Short Period
- Manual Scram

<u>System or Function Undergoing Test</u>	<u>Frequency of Routine Tests</u>	<u>Reference Procedure Within These Specifications</u>
Containment sphere access airlocks leakage rate	6 months or less	
Control rod performance	At each major refueling shutdown**	Section 5.2.2
Liquid poison system component operability	Two months or less during power operation. One equalizing line squib and one remaining squib assembly test fired at least every 18 months.	Section 5.2.3
Reactor safety system scram circuits requiring plant shutdown to check	At each major refueling shutdown*	Section 6.1.5
Reactor safety system scram circuits not requiring plant shutdown to check	One month or less	Section 6.1.5
Containment sphere isola- tion trip circuits	At each major refueling shutdown*	Section 6.1.5
Emergency Condenser Trip Circuits	At each major refueling shutdown*	Section 6.1.5
Control rod withdrawal per- missive interlocks function	Prior to each major refueling shutdown*	Section 6.2.2
Refueling operation controls function	At each major refueling shutdown	Section 6.3.3
Calibration and functional test of air ejector off-gas and stack-gas monitors	Per Table 13-2	Section 13.2
Calibration of emergency condenser vent monitors	One month or less	Section 6.4.1
Calibration of process liquid monitors	Per Table 13-2	Section 13.2

*But no less than once every 18 months

**But no less than once every 20 months