



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
NORTHERN STATES POWER COMPANY
PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-282 AND 50-306
CONFORMANCE TO REGULATORY GUIDE 1.97

INTRODUCTION AND SUMMARY

Northern States Power Company (NSP) was requested by Generic Letter 82-33 to provide a report to the NRC describing how the post-accident monitoring instrumentation meets the guidelines of Regulatory Guide 1.97 as applied to emergency response facilities. The licensee responded to the generic letter on April 15, 1983. Response specific to Regulatory Guide 1.97 was provided on September 15, 1983. Additional information was provided by letters dated January 18, 1985 and June 6, 1985.

A detailed review and technical evaluation of the licensee's submittals was performed by EG&G Idaho, Inc., under contract to the NRC, with general supervision by the NRC staff. This work was reported by EG&G in their Technical Evaluation Report (TER), "Conformance to Regulatory Guide 1.97, Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2," dated August 1985 (attached). We have reviewed this report and concur with the conclusion that the licensee either conforms to, or is justified in deviating from, the guidance of Regulatory Guide 1.97 for each post-accident monitoring variable.

EVALUATION CRITERIA

Subsequent to the issuance of the generic letter, the NRC held regional meetings in February and March 1983 to answer licensee and applicant questions and concerns

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regarding the NRC policy on Regulatory Guide 1.97. At these meetings, it was noted that the NRC review would only address exceptions taken to the guidance of Regulatory Guide 1.97. Further, where licensees or applicants explicitly state that instrument systems conform to the provisions of the regulatory guide, it was noted that no further staff review would be necessary. Therefore, the review performed and reported by EG&G only addresses exceptions to the guidance of Regulatory Guide 1.97. This Safety Evaluation addresses the licensee's submittals based on the review policy described in the NRC regional meetings and the conclusions of the review as reported by EG&G.

EVALUATION

We have reviewed the evaluation performed by our consultant contained in the enclosed TER and concur with its bases and findings. The licensee either conforms to, or has provided an acceptable justification for deviations from the guidance of Regulatory Guide 1.97 for each post-accident monitoring variable.

CONCLUSION

Based on the staff's review of the enclosed Technical Evaluation Report and the licensee's submittals, the staff finds that the Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2, design is acceptable with respect to conformance to Regulatory Guide 1.97, Revision 2.

Principal Contributor:
J. Joyce

Attachment: EG&G Idaho Technical
Evaluation Report

Date: October 18, 1985