NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Dresden Nuclear Power Station, Unit 2	0 5 0 0 0 2 3 7	1 OF 0 2

TITLE (4)
Refuel Floor High Radiation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									Dresden Unit 3	0 5 0 0 0 2 4 9
0 7	1 6	8 5	8 5	- 0 3 0	- 0 0	0 7	3 1	8 5	N/A	0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 9 8		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Jerry F. Lizalek (X421)	8 1 5 9 4 2 2 9 2 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X				N							

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Unit 2 was at steady power on 7/16/85 when a high radiation signal from the refueling floor radiation monitors caused a Unit 2 reactor building ventilation system trip and isolation with the standby gas treatment system automatically starting. The Unit 3 reactor building ventilation system was immediately tripped and isolated per DOA 902(3)-3C-16.

This event was caused by the Fuel Handlers lifting machinery from the fuel pool for repairs. Before lifting of the machinery began, the Shift Engineer and Radiation Department were notified of possible high dose levels. As the machine was lifted from the fuel pool, the RCT present registered dose levels of 253R at 6 inches. The machine was immediately submerged and further cleaning performed. When movement was completed, all systems were returned to normal. All personnel working in the area were wearing proper dosimetry, and the dose they received was minimal. This event was of minimal safety significance since all systems functioned as required by Dresden Technical Specification 3.7.B.1. Previous occurrence reported by LER #12-3-85-39.

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Dresden Nuclear Power Station	0 5 0 0 0 2 3 7	8 5	— 0 3 0	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During normal unit operation a high radiation signal from the refuel floor radiation monitors resulted in a Unit 2 reactor building ventilation system trip and isolation. This caused the Standby Gas Treatment System to automatically start. Immediately, the NSO tripped and isolated the Unit 3 reactor building ventilation system per DOA 902(3)-3C-16, resulting in all ventilation exhausting through the Standby Gas Treatment System.

This event was caused by the Fuel Handlers lifting a control rod blade cutting machine from the fuel pool for repairs. Prior to lifting the machine, the Shift Engineer and Radiation Department were notified of possible high dose levels. As the machine was lifted from the fuel pool, the RCT present registered dose levels of 253R at 6 inches. The machine was immediately submerged, and further cleaning performed. It was discovered that boron tubes, approximately 6 inches in length, were wedged in the machine concealed from the Fuel Handler's view. When movement of the machinery was completed, all systems were returned to normal. All personnel working in the area were wearing proper dosimetry and the dose they received was minimal. This event was of minimal safety significance since all systems functioned as required by Dresden Technical Specification 3.7.B.1. Previous occurrence reported by LER #12-3-85-39.

Commonwealth Edison
Dresden Nuclear Power Station
R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

July 31, 1985

DJS LTR: #85-784

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #85-030-0, Docket #050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(a)(2)(iv).

D. J. Scott
Station Manager

DJS/rme

Enclosure

cc: J. G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

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