

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Millstone Nuclear Power Station Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 2 4 5 1 OF 0 2										PAGE 15																											
TITLE (4) Automatic Actuation of an Engineered Safety Feature																																															
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)																													
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES						DOCKET NUMBER(S)														
0 9			1 6			8 5			8 5			-			0 1			6			-			0 0			1 0			1 6			8 5			0 5 0 0 0						0 5 0 0 0					
OPERATING MODE (9)						THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5 (Check one or more of the following) (11)																73.71(b)																									
POWER LEVEL (10) 1 1 0 1 0						20.402(b)						20.405(e)						<input checked="" type="checkbox"/> 80.73(a)(2)(iv)						73.71(c)																							
						20.405(a)(1)(i)						80.36(e)(1)						80.73(a)(2)(v)						73.71(d)																							
						20.405(a)(1)(ii)						80.36(e)(2)						80.73(a)(2)(vi)						OTHER (Specify in Abstract below and in Text, NRC Form 366A)																							
						20.405(a)(1)(iii)						80.73(a)(2)(i)						80.73(a)(2)(vii)(A)																													
						20.405(a)(1)(iv)						80.73(a)(2)(ii)						80.73(a)(2)(vii)(B)																													
20.405(a)(1)(v)						80.73(a)(2)(iii)						80.73(a)(2)(viii)																																			
20.405(a)(1)(vi)						80.73(a)(2)(iv)						80.73(a)(2)(ix)																																			
20.405(a)(1)(vii)						80.73(a)(2)(v)						80.73(a)(2)(x)																																			
LICENSEE CONTACT FOR THIS LER (12)																																															
NAME David J. Yapchanyk, Engineer																TELEPHONE NUMBER AREA CODE 2 1 0 3 4 4 7 1 - 1 7 1 9 1 1																															
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																															
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC																																					
				N																																											
SUPPLEMENTAL REPORT EXPECTED (14)																EXPECTED SUBMISSION DATE (15)						MONTH DAY YEAR																									
YES (If yes, complete EXPECTED SUBMISSION DATE)																<input checked="" type="checkbox"/> NO																															

ABSTRACT (Limit to 1400 spaces - i.e., approximately fifteen single-space typewritten lines) (16)

On September 16, 1985 at 1940 hours, while operating at 100% power, an automatic actuation of an engineered safety feature occurred. A high radiation spike, resulting from placing a condensate demineralizer inservice, caused the reactor building ventilation to isolate and the standby gas treatment to initiate. The isolation signal was reset and the reactor building ventilation and the standby gas treatment system returned to normal at 2000 hours.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Millstone Nuclear Power Station Unit 1		05	000245	85-016	-00	02	OF 02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On September 16, 1985 at 1940 hours while operating at 100% power, an automatic actuation of an engineered safety feature occurred. A high radiation spike, resulting from placing a condensate demineralizer inservice, caused the reactor building ventilation to isolate and the standby gas treatment to initiate. The isolation signal was reset and the reactor building ventilation and the standby gas treatment system returned to normal at 2000 hours.

The isolation of the reactor building ventilation and initiation of the standby gas treatment resulted from a steam tunnel high radiation spike. The high radiation spike in the steam tunnel has been attributed to a corrosion product release which was introduced to the reactor upon valving in a recycled condensate demineralizer. High corrosion product levels have been observed in condensate demineralizers and are the result of a reduced ultrasonic resin cleaning (URC) frequency.

The chemistry department has returned to a normal URC configuration. The chemistry department procedures have been revised ensuring at URC's are conducted on a frequency which will preclude the recurrence of this event. The procedures also offer direction on power level limitations should demineralizer differential pressure become excessive. High radiation spikes have not been observed since the normal URC configuration has been in effect.

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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October 16, 1985
MP-8270

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D C. 20555

Reference: Facility Operating License No. DPR-21
Docket No. 50-245
Reportable Occurrence RO-85-016-00

Gentlemen:

This letter forwards the Licensee Event Report 85-016-00 required to be submitted within thirty days pursuant to the requirements of 10CFR50.73.

Very truly,

NORTHEAST NUCLEAR ENERGY COMPANY

Wayne D. Romberg

Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

WDR/DY:ejl

Attachment: LER 85-016-00

cc: Dr. T. E. Murley, Region I

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