

RANCHO SECO UNIT 1
TECHNICAL SPECIFICATIONS

LIST OF TABLES

<u>Table</u>		<u>Page</u>
2.3-1	REACTOR PROTECTION SYSTEM TRIP SETTING LIMITS	2-9
3.5.1-1	INSTRUMENTS OPERATING CONDITIONS	3-27
3.6-1	SAFETY FEATURES CONTAINMENT ISOLATION VALVES	3-40
3.14-1	FIRE DETECTION INSTRUMENTS FOR SAFETY SYSTEMS	3-55
3.14-2	INSIDE BUILDING FIRE HOSE STATIONS	3-57a
3.15-1	RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION	3-61
3.16-1	RADIOACTIVE GASES EFFLUENT MONITORING INSTRUMENTATION	3-64
3.22-1	RADIOACTIVE ENVIRONMENTAL MONITORING PROGRAM	3-83
3.22-2	REPORTING LEVELS FOR RADIOACTIVITY CONCENTRATIONS IN ENVIRONMENTAL SAMPLES	3-86
4.1-1	INSTRUMENT SURVEILLANCE REQUIREMENTS	4-3
4.1-2	MINIMUM EQUIPMENT TEST FREQUENCY	4-8
4.1-3	MINIMUM SAMPLING FREQUENCY	4-9
110> 4.2-1	THIS TABLE HAS BEEN DELETED	4-12a
4.2-2	INSERVICE INSPECTION SCHEDULE	4-13
4.10-1	ENVIRONMENTAL RADIATION MONITORING PROGRAM	4-42
4.10-2	OPERATIONAL ENVIRONMENTAL RADIATION MONITORING PROGRAM	4-22a
4.14-1	SNUBBERS ACCESSIBLE DURING POWER OPERATIONS	4-47c
4.17-1	MINIMUM NUMBER OF STEAM GENERATORS TO BE INSPECTED DURING INSERVICE INSPECTION	4-56
4.17-2	STEAM GENERATOR TUBE INSPECTION	4-57
4.19-1	RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	4-64
4.20-1	RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS	4-66

Proposed Amendment No. 110, Rev. 2

ix

8510210066 851014
PDR ADOCK 05700312
P PDR

RANCHO SECO UNIT 1
TECHNICAL SPECIFICATIONS

Surveillance Standards

4.2 REACTOR COOLANT SYSTEM SURVEILLANCE

Applicability

Applies to the reactor vessel, the reactor coolant system and its components.

Objective

To establish examinations whereby the reactor coolant system and component integrity is monitored.

Specification

110* 4.2.1 This paragraph has been deleted.

4.2.2 An inservice inspection shall be made conforming as closely as design permits to the rules of the ASME Boiler and Pressure Vessel Code Section XI, Rules for Inservice Inspection of Nuclear Reactor Coolant Systems with revisions approved as of June 1973, tables IS-261, IS-251 and IS-240 of this Code, will be used as a guide for determining the examination frequencies and the applicable specific areas to be examined. The inspection interval will be ten years. As part of the inservice inspection, hydrostatic tests will be performed as prescribed under Section IS-500 of this Code.

4.2.3 A preoperational examination will be made to include all the items that would normally be completed throughout the inspection interval. This survey will establish initial system integrity and provide a baseline for future testing.

4.2.4 Each reactor coolant pump motor flywheel will be inspected volumetrically during the ten-year inspection interval. One hundred percent of the flywheel will be examined. All flywheels received a one hundred percent ultrasonic examination prior to installation on the motor.

Because the reactor coolant system was not designed to meet the requirements of Section XI of the ASME Boiler and Pressure Vessel Code, complete compliance is not feasible or practical. However, access for inservice inspection has been considered and design modifications made where practical.

RANCHO SECO UNIT 1
TECHNICAL SPECIFICATIONS

Surveillance Standards

4.2.4 (Continued)

Therefore, where possible, Section XI of this Code will be utilized in the conduct of this program. Table 4.2-2 itemizes those areas where complete compliance with the code is not possible because of specific design and construction details.

4.2.5 If as a result of any of these inspections, defects are found to develop, further examinations will be made as needed to determine the exact condition. Following evaluation of this evidence, a decision will be made to the effect upon plant safety and the requirements for repairs.

4.2.6 Records of each inspection shall be kept to permit evaluation and future comparison.

4.2.7 Periodic consideration will be given to incorporation of new or improved inspection techniques into the surveillance program.

110> 4.2.8 This paragraph has been deleted.

Bases

110> This paragraph has been deleted.

RANCHO SECO UNIT 1
TECHNICAL SPECIFICATIONS

Surveillance Standards

Preoperational and inservice inspections emphasize areas of highest stress concentration and probability of failure. The area predominantly selected for these examinations are welds and the adjacent metal. Examination of the welds is often by a volumetric (ultrasonic or radiography) method which, when performed, examines surrounding base metal and the weld heat-affected zone. Both testing methods will use present state-of-the-art equipment operated by highly trained personnel qualified within the requirements of the applicable codes.

110>< This paragraph has been deleted.

RANCHO SECO UNIT 1
TECHNICAL SPECIFICATIONS

Surveillance Standards

Table 4.2-1

110>€

THIS TABLE HAS BEEN DELETED