

NRC FORM 366
(12-81)
19 CFR 50.U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORTAPPROVED BY OMB
3150-0011

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 PASES 100-000000-0004111145
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

CONT

01 REPORT SOURCE L605000387702228380716859
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58 59 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80 81 82 83 84 85 86 87 88 89 90 91 92 93 94 95 96 97 98 99 100

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 While in Operating Condition 2, startup, both channels of Primary Containment

03 Atmosphere Gaseous Radioactivity Monitoring were inoperable (Technical Specifica-

04 tion 3.4.3.1). The indicating meter in the "A" channel monitor was reading down

05 scale and the flow indicator on the "B" indicated no flow. There were no con-

06 sequential effects to the public health and safety. The primary coolant leakage

07 was being monitored by a particulate monitoring channel and two drywell sump

08 level monitoring channels. All monitoring means indicated normal leakage.

09 SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP. SUBCODE VALVE SUBCODE

C I 11 B 12 A 13 I N S T R U 14 I 15 Z 16

17 LER/RO REPORT NUMBER 83 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32

ACTION TAKEN FUTURE ACTION EFFECT ON PLANT SHUTDOWN METHOD HOURS 22 ATTACHMENT SUBMITTED NPD-4 FORM SUB. PRIME COMP. SUPPLIER COMPONENT MANUFACTURER 26

F 18 X 19 Z 20 Z 21 00000 N 23 N 24 A 25 N 30 5

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The "A" channel meter was apparently stuck down scale. A mechanical shock freed

11 up the meter and a channel check proved it operable. The "B" channel was found

12 to have very little flow. When the system's common sample return line was blown

13 down, it was determined to have been blocked by particulate matter. Refer to LER

14 83-100/03X-1 for discussion of other system problems.

15 FACILITY STATUS % POWER OTHER STATUS 30 METHOD OF DISCOVERY DISCOVERY DESCRIPTION 32

C 28 000 29 NA B 31 Operator observation

16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36

Z 33 Z 34 NA NA

17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION 39

000 37 Z 38 NA

18 PERSONNEL INJURIES NUMBER DESCRIPTION 41

000 40 NA

19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION 43

Z 42 NA

20 PUBLICITY ISSUED DESCRIPTION 45

N 44 NA

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PDR ADOCK 05000387
S PDRIE22
1/1

NAME OF PREPARER L.A. Kuczynski

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PP&L

July 16, 1985

SUSQUEHANNA STEAM ELECTRIC STATION
PO BOX 467, BERWICK, PA 18603

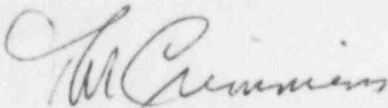
Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 83-042/03X-1
ER 100450 FILE 841-23
PLAS- 102

Docket No. 50-387
License No. NPF-14

Dear Dr. Murley:

Attached is updated Licensee Event Report No. 83-042/03X-1. This event was determined to be reportable per Technical Specification 6.9.1.9.b, in that two channels of Primary Containment Atmosphere Gaseous Radioactivity Monitoring were inoperable at the same time.



T.M. Crimmins
Superintendent of Plant-Susquehanna

LAK/pjg

cc: Mr. R.H. Jacobs
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