NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Browns Ferry - Unit 1	0 5 0 0 0 2 5 9	1 OF 0 2

TITLE (4)
Ongoing 10 CFR 50 Appendix J Reviews

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										Browns Ferry - Unit 2	0 5 0 0 0 2 6 0
0 6	2 6	8 5	8 5	-	0 3 6	- 0 0	0 7	2 3	8 5	Browns Ferry - Unit 3	0 5 0 0 0 2 9 6

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	X 50.73(a)(2)(v)	73.71(c)
0 0 0		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Kevin R. Mulling	2 0 5 7 2 9 - 3 8 3 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 26, 1985, during a continuing engineering evaluation of the Appendix J program, it was discovered that additional reactor core isolation cooling (RCIC) and high pressure coolant injection (HPCI) system isolation valves were not being local leak tested (Type C) in accordance with 10 CFR 50, Appendix J requirements. The RCIC and HPCI pump suction isolation valves (FCVs 71-17/18, 73-26/27) and mini-flow bypass isolation valves (FCVs 71-34/547, 73-30/559) have been added to the Appendix J program and will be Type C tested as required.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Browns Ferry - Unit 1	0 5 0 0 0 2 5 9	8 5	— 0 3 6	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Units 1 and 2 were in a refueling outage, and unit 3 was in an extended maintenance outage. All three units were affected.

During an ongoing engineering evaluation of the Appendix J program on June 26, 1985, it was discovered that the reactor core isolation cooling (RCIC) (BN) and high pressure cooling injection (HPCI) (BG) pump (P) suction isolation valves (ISV) (FCVs 71-17/18, 73-26/27) and mini-flow bypass isolation valves (V) (FCVs 71-34/547, 73-30/559) were not being local leak tested (Type C) as required.

As part of the engineering evaluation, a design study was conducted that identified the subject valves as primary containment (BD) isolation valves. When the Appendix J program was originally developed these valves were considered to be system isolation valves only and not installed to perform a primary containment isolation function, and therefore, were not part of the Appendix J program. In order to prevent recurrence of this problem, the previously mentioned engineering evaluation is being continued and will evaluate all valves that could possibly have test requirements imposed on them by 10 CFR 50, Appendix J.

The HPCI and RCIC system isolation valves identified above have been added to Surveillance Instruction 4.7.A.2.g-3, "Containment Isolation Valve Leak Test." The unit 3 valves have been Type C tested with satisfactory results. The unit 1 and 2 valves will be Type C tested prior to startup. There was no effect on the health or safety of the public. The results of the latest integrated leak rate tests show that primary containment integrity leakage limits were not exceeded.

Responsible Plant Section - N/A

Previous Events - BFRO-50-260/83005; -259/85008

TENNESSEE VALLEY AUTHORITY
Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

July 23, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 1 - DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE REPORT BFRO-50-259/85036

The enclosed report provides details concerning ongoing 10 CFR 50, Appendix J reviews. This report is submitted in accordance with 10 CFR 50.73(a)(2)(v).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

Robert L Lewis

R. L. Lewis
Acting Plant Manager
Browns Ferry Nuclear Plant

Enclosures
cc (Enclosures):
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U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
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Atlanta, Georgia 30303

NRC Resident Inspector, BFN

INPO Records Center
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