

LICENSEE EVENT REPORT (LER)

Facility Name (1) Byron, Unit 1 Docket Number (2) 0 5 0 0 0 4 5 4 Page (3) 1 of 0 2

Title (4) REACTOR TRIP

| Event Date (5) | | | LER Number (6) | | Report Date (7) | | | Other Facilities Involved (8) | |
|----------------|-----|------|----------------|-------------------|-----------------|-------|-----|-------------------------------|----------------|
| Month | Day | Year | Year | Sequential Number | Revision Number | Month | Day | Year | Facility Names |
| 0 6 | 2 4 | 8 5 | 8 5 | 0 6 1 | 0 0 | 0 7 | 2 4 | 8 5 | None |

OPERATING MODE (9) 1 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10CFR (Check one or more of the following) (11)

| | | | | | |
|-------------------------------|--------------------------|-------------------------|----------|-----------------------------|-----------------------|
| POWER LEVEL (10) <u>0 9 7</u> | <u>20.402(b)</u> | <u>20.405(c)</u> | <u>X</u> | <u>50.73(a)(2)(iv)</u> | <u>73.71(b)</u> |
| | <u>20.405(a)(1)(i)</u> | <u>50.36(c)(1)</u> | | <u>50.73(a)(2)(v)</u> | <u>73.71(c)</u> |
| | <u>20.405(a)(1)(ii)</u> | <u>50.36(c)(2)</u> | | <u>50.73(a)(2)(vii)</u> | <u>Other (Specify</u> |
| | <u>20.405(a)(1)(iii)</u> | <u>50.73(a)(2)(i)</u> | | <u>50.73(a)(2)(viii)(A)</u> | <u>in Abstract</u> |
| | <u>20.405(a)(1)(iv)</u> | <u>50.73(a)(2)(ii)</u> | | <u>50.73(a)(2)(viii)(B)</u> | <u>below and in</u> |
| | <u>20.405(a)(1)(v)</u> | <u>50.73(a)(2)(iii)</u> | | <u>50.73(a)(2)(x)</u> | <u>Text)</u> |

LICENSEE CONTACT FOR THIS LER (12)

Name Richard M. Williams, System Test Engineer, Ext. 2385 TELEPHONE NUMBER 8 1 5 2 3 4 - 5 4 4 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFAC-TURER | REPORTABLE TO NPRDS | CAUSE | SYSTEM | COMPONENT | MANUFAC-TURER | REPORTABLE TO NPRDS |
|-------|--------|-----------|---------------|---------------------|-------|--------|-----------|---------------|---------------------|
| A | S J | | | N | | | | | |

SUPPLEMENTAL REPORT EXPECTED (14)

X Yes (If yes, complete EXPECTED SUBMISSION DATE) NO Expected Submission Date (15) 1 0 3 1 8 5

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

While operating in Mode 1 at a reactor power of 97%, a reactor trip occurred due to 1A Steam Generator LO-2 level. The low Steam Generator level was caused by the trip of the 1C Feedwater Pump, after an Equipment Attendant inadvertently activated the local overspeed trip bar. Following the 1C Feedwater Pump trip, the main turbine/generator load was run back and the 1A Motor Driven Feedwater Pump was started. However, the recovery attempt was not successful. This was an isolated incident. The trip lever handles were removed to prevent recurrence. An inspection will be initiated to identify other equipment with similar problems.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

| FACILITY NAME (1) | DOCKET NUMBER (2) | LER NUMBER (6) | | | | | | Page (3) | | | |
|-------------------|-------------------------------|----------------|-----|----------------------|-----|--------------------|-------|----------|-------|--|--|
| | | Year | /// | Sequential Number | /// | Revision Number | | | | | |
| Byron, Unit 1 | 0 5 0 0 0 4 5 4 | 8 5 | - | 0 6 1 | - | 0 0 | 0 2 | OF | 0 2 | | |

TEXT

On June 24th at 0127 CST, Unit 1 was operating in Mode 1 at 97% power. The 1B and 1C Turbine Driven Feedwater Pumps were supplying Steam Generator feedwater demand. The 1A Motor Driven Feedwater Pump was shut down, but available.

The Unit 1 Turbine Building Equipment Attendant was instructing two new trainees at the 1C Turbine Driven Feedwater Pump. The trainees were logging the necessary readings while the Equipment Attendant explained to them what they were recording. As the Equipment Attendant illustrated the last reading, he turned, snagging his 2-way radio microphone cord on the local turbine overspeed trip bar causing the 1C Turbine Driven Feedwater Pump to trip.

Noting the trip of the 1C Feedwater Pump, the Control Room operator immediately started the 1A Motor Driven Feedwater Pump and decreased the Main Turbine/Generator load. Even though turbine and reactor power levels were reduced, a reactor trip occurred due to 1A Steam Generator LO-2 level. Both Auxiliary Feedwater Pumps automatically started to insure minimum Steam Generator levels were maintained. No Safety Injection occurred and the plant was stable in Hot Standby 40 minutes into the event.

This event has never happened before at Byron Station. The event posed no threat to public safety. The licensed personnel on shift and the Reactor Protection Systems reacted appropriately to bring this event to a safe conclusion. Immediate action taken was to remove the trip lever handles. An inspection will be initiated to identify other equipment with similar problems.



Commonwealth Edison
Byron Nuclear Station
4450 North German Church Road
Byron, Illinois 61010

July 24, 1985

LTR: BYRON 85-1051

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

The enclosed Licensee Event Report from Byron Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73(a)(2)(iv) 50.73(a)(2)(i) which requires a 30 day written report.

This report is number 85-061-00; Docket No. 50-454.

Very truly yours,

R. E. Querio
Station Superintendent
Byron Nuclear Power Station

REQ/gt

Enclosure: Licensee Event Report No. 85-061-00

cc: J. G. Keppler, NRC Region III Administrator
J. Hinds, NRC Resident Inspector
INPO Record Center
CECO Distribution List

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