

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Waterford 3 Steam Electric Station										DOCKET NUMBER (2) 0 5 0 0 0 3 8 2										PAGE (3) 1 OF 3				
TITLE (4) Automatic Actuation of Reactor Protective System																								
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)												
0	6	2	7	8	5	8	5	0	2	8	0	0	0	7	2	6	8	5	N/A	0	5	0	0	0
OPERATING MODE (9) 2			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																					
POWER LEVEL (10) 0 0 0			20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)									
			20.405(a)(1)(i)				50.36(c)(1)				<input type="checkbox"/> 50.73(a)(2)(v)				73.71(c)									
			20.405(a)(1)(ii)				50.36(c)(2)				<input type="checkbox"/> 50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
			20.405(a)(1)(iii)				50.73(a)(2)(i)				<input type="checkbox"/> 50.73(a)(2)(viii)(A)													
			20.405(a)(1)(iv)				50.73(a)(2)(ii)				<input type="checkbox"/> 50.73(a)(2)(viii)(B)													
			20.405(a)(1)(v)				50.73(a)(2)(iii)				<input type="checkbox"/> 50.73(a)(2)(x)													
LICENSEE CONTACT FOR THIS LER (12)																								
NAME L. Myers, Operations Superintendent												TELEPHONE NUMBER 5 0 4 4 6 4 - 3 1 1 8												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																								
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS														
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR								
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At 2136 hours on June 27, 1985, Waterford 3 Steam Electric Station was increasing reactor power to 5E-4% for the compilation of criticality data. Since the Plant Monitoring Computer was down, Plant Operators, while attempting to stabilize power at 5E-4%, could only move the regulating group Control Element Assemblies in the manual individual mode. At 2157 hours, while at 8E-4% reactor power, when the second Control Element Assembly was inserted, the Core Protection Calculator, due to the Control Element Assembly position deviation, initiated a large enough penalty factor to generate a DNBR and Local Power Density trip. Plant Conditions were stabilized in mode 3 (hot standby).

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Waterford 3 Steam Electric Station	0 5 0 0 0 3 8 2	8 5	- 0 2 8	- 0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

At 2020 hours on June 27, 1985, Waterford 3 Steam Electric Station began an approach to criticality when the Plant Monitoring Computer crashed. Plant Operators reached criticality at 2136 and began increasing reactor power to 5E-4% in order to compile criticality data as required by procedure OP-10-001, General Plant Operations. As power increased to 2E-4%, Plant Operators attempted to insert the regulating group Control Element Assemblies in the manual group mode in order to stabilize power at 5E-4% for the above testing. However, the regulating Control Element Assemblies would not move in the manual group mode since the lower group stop was actuated due to the loss of the Plant Monitoring Computer. (Since the Plant Monitoring Computer performs the Control Element Assemblies motion permissive calculations, the loss of the Plant Monitoring Computer will prevent the regulating group Control Element Assemblies from moving in the automatic sequential, manual sequential, and manual group mode; however, the Control Element Assemblies can be moved in the manual individual mode. Upon restoring the Plant Monitoring Computer, the Control Element Assemblies are assumed to be at the bottom of the core until the correct position is manually inputted into the computer.) Operations Personnel began inserting the regulating group Control Element Assemblies individually approximately five inches each. At 2157 hours, when the second control Element Assembly was inserted, the Core Protection Calculators, due to the Control Element Assembly position deviation, initiated a large enough penalty factor to generate a DNBR and Local Power density trip. Operations Personnel immediately entered procedures OP-902-000, Emergency Entry Procedure, and OP-902-001, Uncomplicated Reactor Trip Recovery. Plant conditions were stabilized in mode 3 (hot standby).

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in an actuation of the Reactor Protective System during initial Startup testing in which no primary system parameters were exceeded. Since the Control Element Assemblies and the Reactor Protective System functioned as designed, the event in no way placed Waterford 3 in a degraded safety condition.

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		8 5	— 0 2 8	— 0 0	0 3	OF	0 3

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CORRECTIVE ACTION

In order to prevent this condition from recurring, this Licensee Event Report, along with the penalty factors associated with Control Element Assembly position deviations, will be emphasized during operator training.

SIMILAR EVENTS

None

PLANT CONTACT

L. Myers, Operations Superintendent, 504/464-3118



LOUISIANA
POWER & LIGHT / INTER-OFFICE CORRESPONDENCE

July 26, 1985

W3P85-1438
A4.05

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-028-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(iv).

Very truly yours,

A handwritten signature in cursive script that reads 'K.W. Cook'.

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, D.M. Crutchfield, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), B.W. Churchill,
W.M. Stevenson

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