

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Waterford 3 Steam Electric Station										DOCKET NUMBER (2) 0 5 0 0 0 3 8 2										PAGE (3) 1 OF 0 3																																																	
TITLE (4) Manual Actuation of Reactor Protective System																																																																					
EVENT DATE (5) 0 6 2 6 8 5										LER NUMBER (6) 8 5 0 2 7 0 0 0 7 2 6 8 5										REPORT DATE (7) 0 7 2 6 8 5										OTHER FACILITIES INVOLVED (8) N/A																																							
OPERATING MODE (9) 1										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) <input checked="" type="checkbox"/> 20.402(b) <input type="checkbox"/> 20.405(c) <input checked="" type="checkbox"/> 50.73(a)(2)(iv) <input type="checkbox"/> 73.71(b) <input type="checkbox"/> 20.405(a)(1)(i) <input type="checkbox"/> 50.36(a)(1) <input type="checkbox"/> 50.73(a)(2)(v) <input type="checkbox"/> 73.71(c) <input type="checkbox"/> 20.405(a)(1)(ii) <input type="checkbox"/> 50.36(a)(2) <input type="checkbox"/> 50.73(a)(2)(vii) <input type="checkbox"/> OTHER (Specify in Abstract below and in Text, NRC Form 368A) <input type="checkbox"/> 20.405(a)(1)(iii) <input type="checkbox"/> 50.73(a)(2)(i) <input type="checkbox"/> 50.73(a)(2)(viii)(A) <input type="checkbox"/> <input type="checkbox"/> 20.405(a)(1)(iv) <input type="checkbox"/> 50.73(a)(2)(ii) <input type="checkbox"/> 50.73(a)(2)(viii)(B) <input type="checkbox"/> <input type="checkbox"/> 20.405(a)(1)(v) <input type="checkbox"/> 50.73(a)(2)(iii) <input type="checkbox"/> 50.73(a)(2)(ix) <input type="checkbox"/>										POWER LEVEL (10) 0 1 9 1 1																																																	
LICENSEE CONTACT FOR THIS LER (12) NAME: L. Myers, Operations Superintendent TELEPHONE NUMBER: 5 1 0 4 4 6 4 - 1 3 1 1 1 8																																																																					
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																																					
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SUPPLEMENTAL REPORT EXPECTED (14) <input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO																				EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR 1 1 1 8																																																	

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At 2126 hours on June 26, 1985, Waterford 3 Steam Electric Station was at 91% reactor power when Operations Personnel received information of a fire in Feedwater Pump A. The pump was secured and reactor power was reduced in order to compensate for the decrease in feedwater flow. An Unusual Event was declared at 2130 hours. At 2131 hours the Control Room was informed by personnel on the scene that the B Feedwater Pump, rather than the A pump, was on fire. Since the A pump was already secured, and since Steam Generator water levels were decreasing, Operations Personnel tripped the main turbine and reactor.

The fire was extinguished by Plant Personnel at 2136 hours and Waterford 3 secured from the Unusual Event shortly thereafter. The fire, started by a small oil leak, was limited to a small portion of the outer wrapping of insulation on the feedwater piping.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Waterford 3 Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 2	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

At 2126 hours on June 26, 1985, Waterford 3 Steam Electric Station was at 91% reactor power when an electrician notified the Control Room of smoke emanating from Main Feedwater Pump A. An operator, along with a Fire Team Leader, was dispatched to the scene. The operator notified the Control Room that the Main Feedwater Pump was on fire. Control Room Operations immediately tripped Feedwater Pump A and began reducing reactor power. The Fire Team Leader requested offsite assistance, which was then relayed to the proper authorities via the Industrial Hotline by the Shift Supervisor at 2130 hours. An Unusual Event was declared at 2130 hours. At 2131 hours the Control Room was notified that the fire was in Main Feedwater Pump B rather than Pump A, as first reported. The Secondary Nuclear Plant Operator immediately tripped the turbine, and since the turbine trip-reactor trip function was enabled, the reactor tripped. Operations Personnel immediately entered procedure, OP-902-000, Emergency Entry Procedure. Since both Feedwater Pumps were secured, Steam Generator water levels decreased until the Emergency Feedwater Actuation Signal Setpoint was reached.

As the initial responder to the scene, the Fire Team Leader proceeded to extinguish the fire (the fire was extinguished at 2136 hours, resulting in the cancellation of the request for offsite assistance). The offsite fire department responded to the site with full equipment; however, due to the cancellation of the request for support, only the department chief entered the Protected Area to establish contact with the Waterford 3 Fire Team Leader. At 2143 hours Waterford 3 secured from the Unusual Event, and plant conditions were stabilized in mode 3 (hot standby).

The fire was limited to a small portion of the outer wrapping of insulation on the feedwater piping; however some damage was evident on a feedwater pump thermocouple and, therefore, required replacement.

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SAFETY CONSEQUENCES AND IMPLICATIONS

Since the fire was limited to the outer wrapping of the feedwater pipe insulation, and since the fire was extinguished expeditiously, the above event did not pose a threat to plant personnel and equipment.

CORRECTIVE ACTION

The oil reservoir is separated into two sections by a baffle plate. The baffle plate extends from the top of the reservoir to some point just below the mid section of the reservoir. Since the oil level in the reservoir was above the lower portion of the baffle (per design), the vapor extractors could not maintain the required vacuum in the inboard section of the oil reservoir. Because of this, an oil leak developed from various seals associated with the B Feedwater Pump. In order to prevent this condition from recurring, a temporary change has been made to both Feedwater Pumps such that both sections of the oil reservoir are interconnected across the top of the baffle plate. This interconnection will allow the vapor extractors to maintain the necessary vacuum in both sections of the oil reservoir.

Under consideration is a permanent modification which will allow Maintenance Personnel to drill holes in the baffle plate in order to accomplish the results described above.

SIMILAR EVENTS

None

PLANT CONTACT

L. Myers, Operations Superintendent, 504/464-3118



LOUISIANA
POWER & LIGHT / INTER-OFFICE CORRESPONDENCE

July 26, 1985

W3P85-1437
A4.05

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-027-00 for the Waterford 3
Steam Electric Station. This Licensee Event Report is submitted per
10CFR50.73(a)(2)(iv).

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, D.M. Crutchfield, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), B.W. Churchill,
W.M. Stevenson