

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Zion Station - Unit One										DOCKET NUMBER (2) 0 5 0 0 0 2 9 5					PAGE (3) 1 OF 0 2										
TITLE (4) Reactor Trip from Low Steam Generator Level																									
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)												
0	6	2	9	8	5	8	5	0	2	4	0	0	0	7	2	6	8	5	0	5	0	0	0		
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)																							
2		20.402(b)				20.405(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)				73.71(b)											
POWER LEVEL (10)		0 0 1				20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)							
		20.405(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 366A)											
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)															
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)															
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(ix)															
LICENSEE CONTACT FOR THIS LER (12)																									
NAME James J. Madden										TELEPHONE NUMBER 3 1 1 2 7 1 4 6 1 - 1 2 1 0 1 8 1 4															
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																									
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
A																									
B	S	B	I	S	V				Yes																
SUPPLEMENTAL REPORT EXPECTED (14)										EXPECTED SUBMISSION DATE (15)			MONTH DAY YEAR												
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)										<input checked="" type="checkbox"/> NO															

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

The unit was in hot standby, preparing to go to Mode 1. Bistables were thrown to permit work on feed flow instrumentation. Level in "D" S/G reached 25% giving Automatic Reactor Trip. EOP-1 was followed and all notification were made.

The trip was caused by operator error, aggravated by equipment failure. A main steam line drain valve was leaking, causing steam generator "D" to lose level. The unit operator was involved in discussion on an ongoing instrument repair to another system at the time.

The main steam drain valve was torqued down properly, and the operator was counselled on the importance of not getting distracted during transient operation.

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05000295  
PDR

JEER

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)  Zion Station, Unit One	DOCKET NUMBER (2)  0 5 0 0 0 2 9 5 8 5 - 0 2 4 - 0 0 0 2 OF 0 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Unit one reactor was in hot standby mode approximately 1% power. The shift was preparing to bring the unit on line and had just latched the turbine at 0407. At 0412 the main steam isolation valves (MSIVs) were opened. The operator noticed a main steam line drain valve 1MOV-MS0172 was showing double indication. This valve is interlocked to close when the MSIV for that loop is opened. An Equipment Attendent was dispatched to investigate and to torque down on the valve to ensure its closed position.

Temperature continued to trend down, so at 0420 the turbine was tripped and attempts were made to put it on turning gear.

Steam Generator "D" had a feed flow channel which was inoperable. Per the operating procedure, its associated bistable had been tripped half an hour earlier. This was a bistable which compared steam flow to feed flow. This comparator bistable in conjunction with a S/G level of 25% generates a reactor trip in anticipation of a loss of heat sink.

While maintaining levels on the generator, the operator became involved with discussions on plans to accomodate instrument maintenance repair on a letdown pressure control valve. Level in steam generator "D" reached 25% and the automatic reactor trip occurred at 0441. All systems performed as expected. Subsequent to the trip, "D" MSIV drain valve was found to have been  $\frac{1}{2}$ " open prior to its being manually torqued closed.

Root cause of the trip was operator error, exaggerated by equipment failure. Operator was counseled on the importance of not allowing himself to be distracted during transient operation.



**Commonwealth Edison**

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July 26, 1985

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

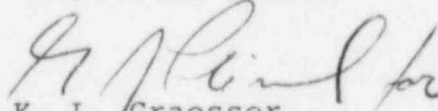
References: 10CFR50

Dear Sir:

The enclosed Licensee Event Report from Zion Generating Station is being transmitted to you in accordance with the requirements of 10CFR50.73 (a)(2)(iv) which requires a 30 day written report when an event or condition results in manual or automatic actuation of any Engineered Safety Feature (ESF), including the Reactor Protection System (RPS).

This report is number 85-024-00, Docket No. 50-295/DPR-39.

Very truly yours,

  
K. L. Graesser  
Station Manager  
Zion Generating Station

KLG/rmm

Enclosure: Licensee Event Report No. 85-024-00

Attachment

cc: J. G. Keppler, NRC Region III Administrator  
M. Holzmer, NRC Resident Inspector  
INPO Record Center  
CECo Distribution List

IE22  
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