

Commonwealth Edison Company  
Braidwood Generating Station  
Route #1, Box 84  
Braceville, IL 60407-9619  
Tel 815-458-2801

**ComEd**

December 13, 1996

United States Nuclear Regulatory Commission  
Attention: Document Control Desk  
Washington, D.C. 20555-0001

Subject: Application for Amendment to Appendix A, Technical Specifications, for  
Facility Operating Licenses:

Byron Nuclear Power Station, Units 1 and 2  
Facility Operating Licenses NPF-37 and NPF-66  
NRC Docket Nos. 50-454 and 50-455

Braidwood Nuclear Power Station, Units 1 and 2  
Facility Operating Licenses NPF-72 and NPF-77  
NRC Docket Nos. 50-456 and 50-457

Conversion to the Improved Standard Technical Specifications

- References:
1. Karl L. Kaup and Kenneth L. Graesser (ComEd) letter to NRC Document Control Desk, "Notification of Intent to Adopt the Improved Standard Technical Specifications," dated May 31, 1995.
  2. NUREG-1431, "Standard Technical Specifications - Westinghouse Plants," Revision 1, dated April 1995.

Ladies and Gentlemen:

Pursuant to Title 10, Code of Federal Regulations, Part 50, Section 90 (10 CFR 50.90), Commonwealth Edison Company (ComEd) proposes to amend Appendix A, Technical Specifications, for Facility Operating Licenses NPF-37, NPF-66, NPF-72, and NPF-77 for Byron Nuclear Power Station, Units 1 and 2 (Byron), and Braidwood Nuclear Power Station, Units 1 and 2 (Braidwood), respectively.

9701060043 961213  
PDR ADOCK 05000454  
P PDR

*Handwritten notes:*  
Acol  
11/11  
Limited  
Dist  
Encls To:  
FM  
Files  
PDR

Reference 2). The proposed license amendment request to convert the Byron and Braidwood CTS to the Byron and Braidwood Improved Technical Specifications (ITS) is enclosed with this letter. The proposed license amendment request was prepared considering the guidance of Nuclear Energy Institute (NEI) NEI 96-06, "Improved Technical Specifications Conversion Guidance," dated August 1996. NEI 96-06 was issued after the start of the ITS project. The detailed description and justification of this proposed license amendment request consists of seventeen (17) volumes. A detailed description of the contents and organization of the 17 volumes is included in Attachments 1 through 7 of this letter, and are described below:

Attachment 1, "Synopsis of the Proposed License Amendment Request," describes the organization and content of the submittal.

Attachment 2, "Differences between Byron and Braidwood," lists the Technical Specification differences between Byron and Braidwood. This attachment is provided to facilitate NRC review.

Attachment 3, "Existing and Future License Amendment Requests to be Incorporated into the ITS," provides a listing of all currently docketed license amendment requests that have been incorporated into this license amendment request. The currently docketed license amendment request regarding boraflex has not been incorporated into this license amendment request pending final resolution of the issue. Additionally, a license amendment request to implement the Pressure/Temperature Limits Report (PTLR) will be made in the near future and is required to support this license amendment request.

Attachment 4, "Pending and Proposed ISTS Change Travelers," provides a listing of the pending and proposed changes to the ISTS incorporated into this license amendment request.

Attachment 5, "'Beyond Scope' Changes," provides a listing of those changes that are different than both CTS and ISTS.

Attachment 6, "'Beyond Scope Bracketed' Changes," provides a listing of those changes to CTS parameters beyond those needed to conform with the ITS.

Attachment 7, "CTS Requirements Relocated to Other Licensee Controlled Documents," provides a listing of the CTS requirements relocated to licensee controlled documents, the proposed location of the relocated item, and the change control mechanism that will be applied to the relocated item.

Implementation of the Byron and Braidwood ITS will require the performance of a number of new surveillance requirements. ComEd intends to treat these new requirements as being "met" at the time of implementation, with the first performance of these new surveillance requirements scheduled to be completed within the required frequency from the date of implementation.

December 13, 1996

Any revisions to the Byron and Braidwood Updated Final Safety Analysis Report (UFSAR) required as a result of this proposed license amendment request will be made in accordance with 10 CFR 50.71(e).

Currently, ComEd intends to implement this proposed license amendment request at both Byron and Braidwood in November 1997. This date is based on the time required for procedure revisions, including the development of new programs, training schedules for both licensed and non-licensed operators, and the timing of implementation with respect to current outage schedules. This date is also predicated on the NRC review being completed and a Safety Evaluation issued by July 1997. However, the implementation date at Byron is tentative and dependent upon final determination of the steam generator replacement outage schedule.

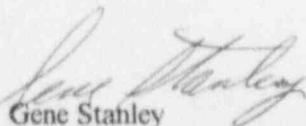
The proposed changes in this license amendment request have been reviewed and approved by both On-site and Off-site Review in accordance with ComEd procedures. ComEd has reviewed this proposed license amendment request in accordance with 10 CFR 50.92(c) and has determined that no significant hazards consideration exists. An environmental assessment has been completed and is included in Attachment 8.

ComEd is notifying the State of Illinois of our application for this license amendment request by transmitting a copy of this letter and its attachments to the designated State Official.

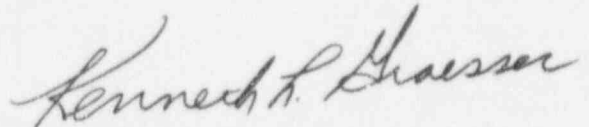
To the best of my knowledge and belief, the statements contained in this document are true and correct. In some respects these statements are not based on my personal knowledge, but on information furnished by other ComEd employees, contractor employees, and/or consultants. Such information has been reviewed in accordance with company practice, and I believe it to be reliable. ComEd requests a meeting at your earliest convenience to discuss a review schedule and contents of this license amendment request. Please address any comments or questions regarding this matter to Mr. Harold D. Pontious, Jr., Braidwood Nuclear Licensing Administrator, at (630) 663-7205.

Very truly yours,

Very truly yours,



Gene Stahley  
Site Vice President  
Braidwood Nuclear Power Station



Ken Graesser  
Site Vice President  
Byron Nuclear Power Station

December 13, 1996

Signed before me

on this 12 dayof December 1996

OFFICIAL SEAL  
TINA M TAMAYO-SANTOLIN  
NOTARY PUBLIC, STATE OF ILLINOIS  
MY COMMISSION EXPIRES 04/19/00  
by Tina M. Tamayo-Santolin  
Notary Public

Signed before me

on this 13<sup>th</sup> dayof December 1996

OFFICIAL SEAL  
JACQUELINE T EVANS  
NOTARY PUBLIC, STATE OF ILLINOIS  
MY COMMISSION EXPIRES 12/15/97  
by Jacqueline T. Evans  
Notary Public

Attachments (8)

Enclosure (1)

cc. A. B. Beach, Regional Administrator - RIII  
C. I. Grimes, Chief, Technical Specifications Branch - NRR  
G. F. Dick Jr., Byron Project Manager - NRR  
R. R. Assa, Braidwood Project Manager - NRR  
S. D. Burgess, Senior Resident Inspector - Byron  
C. J. Phillips, Senior Resident Inspector - Braidwood  
Office of Nuclear Facility Safety - IDNS



ATTACHMENT 1

SYNOPSIS OF THE  
PROPOSED LICENSE AMENDMENT REQUEST

**SYNOPSIS OF THE IMPROVED TECHNICAL SPECIFICATIONS SUBMITTAL**  
(page 1 of 5)

The submittal for the conversion to Improved Technical Specifications (ITS) consists of 17 volumes and related attachments to the transmittal letter. The 17 volumes consist of the application of NRC Selection Criteria (Split Report) and ITS Section packages. Below is a brief description of the contents of the Split Report and each of the Section packages, as well as a brief explanation of how the material was prepared and the designations utilized.

**APPLICATION OF NRC SELECTION CRITERIA**

The Selection Criteria provides a discussion of how the 10 CFR 50.36(c)(2)(ii) criteria were applied to the Byron and Braidwood Current Technical Specifications (CTS) requirements. Also included is a matrix which cross references the following documents: 1) CTS; 2) NUREG-0452, "Standard Technical Specifications for Westinghouse Pressurized Water Reactors," Revision 4; 3) proposed ITS, where applicable; and 4) 10 CFR 50.36(c)(2)(ii) criteria for inclusion of requirements. For those CTS requirements that do not meet any of the NRC selection criteria and are not retained in the proposed ITS, an evaluation of the CTS requirement against the criteria is provided in Appendix A (WCAP-11618 Evaluated) and Appendix B (Non WCAP-11618).

**SECTION PACKAGES FOR SECTIONS 1.0 THROUGH 5.0 (14 SECTIONS)**

Each of the Section packages corresponds to a Section of the proposed ITS, contains the required information to review the ITS Section, and is organized as described below:

TAB 1     Byron ITS

Contains the proposed Byron ITS LCOs and Bases.

TAB 2     Braidwood ITS

Contains the proposed Braidwood ITS LCOs and Bases.

**SYNOPSIS OF THE IMPROVED TECHNICAL SPECIFICATIONS SUBMITTAL**  
(page 2 of 5)

TAB 3     Byron CTS Markups, and  
TAB 4     Braidwood CTS Markups

Contains annotated copies of the CTS pages which show the disposition of existing requirements into the proposed ITS. The pages are arranged in CTS order. The upper right hand corner of the CTS page is annotated with all the ITS Section numbers in which the CTS page occurs. Items on the CTS page that are addressed in other proposed ITS sections are annotated with the appropriate location.

Where a proposed ITS requirement differs from a CTS requirement, individual details of the CTS revision are annotated with alpha-numeric designators which relate to the appropriate Discussion of Change (DOC). The DOC provides a concise justification for the change. The DOCs are located behind the fifth Tab (labeled CTS DOCs) of each Section package. The alpha-numeric designators also relate to the evaluations supporting a finding of No Significant Hazards Consideration (NSHC) located behind the tenth Tab (labeled NSHC) of each Section package.

The DOCs are numbered sequentially within each letter category for each ITS Section. The proposed changes for each CTS requirement are separated into the following categories:

<u>Designator</u>	<u>Category</u>
A	ADMINISTRATIVE - changes to the CTS that result in no additional or reduced restrictions or flexibility. These changes are supported in aggregate by a single NSHC.
R	RELOCATIONS - changes to the CTS that encompass the requirements that do not meet the selection criteria of 10 CFR 50.36(c)(2)(ii). These changes are supported in aggregate by a single NSHC.
M	MORE RESTRICTIVE - changes to the CTS that result in added restrictions or reduced flexibility. These changes are supported in aggregate by a single NSHC.

**SYNOPSIS OF THE IMPROVED TECHNICAL SPECIFICATIONS SUBMITTAL**  
(page 3 of 5)

TAB 3     Byron CTS Markups, and  
TAB 4     Braidwood CTS Markups  
            (continued)

<u>Designator</u>	<u>Category</u>
LA	LESS RESTRICTIVE "Generic" - changes to the CTS that eliminate detail and relocate the detail to a licensee controlled document. Typically, this involves details of system design and function, or procedural detail on methods of conducting a surveillance. These changes are supported in aggregate by a single NSHC.
L	LESS RESTRICTIVE "Specific" - changes to the CTS that result in reduced restrictions or added flexibility. Each less restrictive change is supported by a corresponding evaluation supporting a finding of NSHC.

The CTS Bases pages are replaced in their entirety. An "A" DOC justifies the replacement.

The CTS pages in the Section packages reflect License Amendments issued as of December 13, 1996, and License Amendment Requests described in Attachment 4 to the submittal letter.

TAB 5     CTS DOCs

Contains the DOCs which describe each proposed change to the CTS.

TAB 6     LCO Markups

Contains annotated copies of the applicable NUREG-1431, Revision 1, LCOs which show how the proposed ITS LCO differs from the NUREG LCO. Where a proposed ITS LCO requirement differs from the NUREG LCO, individual details of the change are annotated with alpha-numeric designators which relate to the appropriate Justification for Difference (JFD). The JFD provides a concise justification for the change. The LCO JFDs are located behind the seventh Tab (labeled LCO JFDs) of each Section package.

**SYNOPSIS OF THE IMPROVED TECHNICAL SPECIFICATIONS SUBMITTAL**

(page 4 of 5)

TAB 6    LCO Markups  
(continued)

The LCO JFDs are numbered sequentially within each letter category for each ITS Section. The differences are separated into the following categories.

<u>Designator</u>	<u>Category</u>
B	BRACKETED CHANGES - changes which reflect plant specific information where the NUREG contains brackets.
C	GENERIC CHANGES - changes which reflect generic modifications to NUREG-1431, Revision 1. These changes have either been approved by the NRC or have been proposed by the NRC or industry. Generic changes that were proposed as of October 2, 1996, either as TSTFs or WOG travelers are addressed. Attachment 2 to the submittal letter provides a summary of the generic change disposition.
P	PLANT SPECIFIC CHANGES - changes which reflect plant specific requirements and information.

TAB 7    LCO JFDs

Contains the JFDs which describe the differences from the NUREG-1431 LCOs.

TAB 8    Bases Markups

Contains annotated copies of the applicable NUREG-1431, Revision 1, Bases which show how the proposed ITS Bases differs from the NUREG Bases. Where a proposed ITS Bases requirement differs from the NUREG Bases, individual details of the change are annotated with alpha-numeric designators which relate to the appropriate JFD. The JFD provides a concise justification for the change. The Bases JFDs are located behind the ninth Tab (labeled Bases JFDs) of each Section package.



**SYNOPSIS OF THE IMPROVED TECHNICAL SPECIFICATIONS SUBMITTAL**  
(page 5 of 5)

TAB 8     Bases Markups  
              (continued)

The Bases JFDs are numbered sequentially within each letter category for each ITS Section. The differences are separated into the following categories.

<u>Designator</u>	<u>Category</u>
B	BRACKETED CHANGES - changes which reflect plant specific information where the NUREG contains brackets.
C	GENERIC CHANGES - changes which reflect generic modifications to NUREG-1431, Revision 1. These changes have either been approved by the NRC or have been proposed by the NRC or industry. Generic changes that were proposed as of October 2, 1996, either as TSTFs or WOG travelers are addressed. Attachment 2 to the submittal letter provides a summary of the generic change disposition.
P	PLANT SPECIFIC CHANGES - changes which reflect plant specific requirements and information.

TAB 9     Bases JFDs

Contains the JFDs which describe the differences from the NUREG-1431 Bases.

TAB 10    NSHC

Contains the evaluations required by 10 CFR 50.91(a) supporting a finding of No Significant Hazard Consideration (NSHC). Generic evaluations for a finding of NSHC have been written for each category of changes except Category "L." The evaluations supporting a finding of NSHC are ordered as follows: A, R, M, LA, and L. Each evaluation is annotated to correspond to the DOC addressed in the NSHC.

Additionally, information in support of the conclusion that this amendment request satisfies the categorical exclusion specified in 10 CFR 51.22(c) to perform an Environmental Assessment or Environmental Impact Statement is provided as Attachment 8 to the submittal letter.

ATTACHMENT 2

DIFFERENCES BETWEEN  
BYRON AND BRAIDWOOD

**DIFFERENCES BETWEEN BYRON AND BRAIDWOOD**  
(page 1 of 4)

ITS SECTION	CTS LCO (Applicable DOC)	ITS LCO AND BASES (Applicable JFD)	DESCRIPTION OF DIFFERENCE
1.0 USE AND APPLICATION	None	None	
2.0 SAFETY LIMITS	SL 2.1.1, Footnotes (DOC A <sub>4</sub> )	Not applicable	Deleted expired cycle specific reactor core safety limits for each station.
3.0 LCO AND SR APPLICABILITY	None	None	
3.1 REACTIVITY CONTROL SYSTEMS	None	None	
3.2 POWER DISTRIBUTION LIMITS	None	None	
3.3 INSTRUMENTATION	Table 2.2-1, Footnotes (DOC A <sub>7</sub> )	Not applicable	Deleted expired Byron cycle specific values.
	Table 3.3-4 (DOC A <sub>7</sub> )	Not Applicable	Deleted expired Braidwood cycle specific values.
	Table 3.3-4 (DOC LA <sub>3</sub> )	Bases 3.3.5	The degraded voltage instrumentation allowable values are different for each station.
	None	Bases 3.3.5	The undervoltage and degraded voltage instrumentation design descriptions are modified to reflect station design differences.
3.4 REACTOR COOLANT SYSTEM	Not applicable	Bases 3.4.1	Byron Unit 2, required minimum Reactor Coolant System flow rate is met with $\geq 92\%$ indicated flow rate rather than the $\geq 95\%$ indicated flow rate for Byron Unit 1 and Braidwood Units 1 and 2.
3.5 EMERGENCY CORE COOLING SYSTEMS	LCO 3.5.1 (DOC A <sub>3</sub> )	Not applicable	Deleted expired cycle specific boron concentration values for each station.
	LCO 3.5.5 (DOC A <sub>17</sub> )	Not applicable	Deleted expired cycle specific boron concentration values for each station.
3.6 CONTAINMENT SYSTEMS	None	None	

**DIFFERENCES BETWEEN BYRON AND BRAIDWOOD**  
(page 2 of 4)

ITS SECTION	CTS LCO (Applicable DOC)	ITS LCO AND BASES (Applicable JFD)	DESCRIPTION OF DIFFERENCE
3.7 PLANT SYSTEMS	SR 4.7.1.1 (DOC A <sub>3</sub> )	Not applicable	Deleted expired Braidwood cycle specific condition of operation.
	Not applicable	Bases SR 3.7.1 (JFD P <sub>3</sub> and P <sub>50</sub> )	Braidwood does not test the Main Steam Safety Valves to the OM-1987 code. Byron IST program references ANSI/ASME OM-1-1987 and applicable addenda.
	LCO 3.7.5 (Various DOCs)	LCO 3.7.9, Bases 3.7.9 (Various JFDs)	Braidwood ultimate heat sink design consists of a cooling pond. The Byron ultimate heat sink design consists of mechanical draft cooling towers.
	SR 4.7.6.e.3 SR 4.7.6.e.5 (DOC A <sub>22</sub> )	Not applicable	Deleted expired Braidwood cycle specific surveillance requirements.
	LCO 3.7.7 (DOC A <sub>30</sub> )	Not applicable	Deleted expired Braidwood time specific Specification exception.
	SR 4.7.7.d.3 (DOC L <sub>27</sub> )	SR 3.7.12.4	Braidwood confirms the ability of the Nonaccessible Area Exhaust Filter Plenum Ventilation System (two trains) to maintain the ECCS equipment rooms at a negative pressure while operating at a specified flow rate through the train and per bank. Byron performs this test while operating at a specified flow rate through the train only.
	SR 4.7.7.d.3 (DOC A <sub>46</sub> )	SR 3.7.12.4 Bases SR 3.7.12.4	The specified nonaccessible Area Exhaust Filter Plenum Ventilation System test flow rates are different for each station.
	SR 4.9.4.2 (DOC A <sub>31</sub> )	Not applicable	Deleted expired Braidwood time specific surveillance requirement exception.

**DIFFERENCES BETWEEN BYRON AND BRAIDWOOD**  
(page 3 of 4)

ITS SECTION	CTS LCO (Applicable DOC)	ITS LCO AND BASES (Applicable JFD)	DESCRIPTION OF DIFFERENCE
3.8 ELECTRICAL POWER SYSTEMS	LCO 3.8.2.1, LCO 3.8.2.2 (DOC A <sub>12</sub> and A <sub>13</sub> )	LCO 3.8.4, LCO 3.8.5 Bases 3.8.4, Bases 3.8.5	The battery types are different for each station. Byron uses Gould batteries and Braidwood uses AT&T batteries.
	LCO 3.8.2.1 LCO 3.8.2.2	LCO 3.8.4, LCO 3.8.5 (JFD P <sub>15</sub> and P <sub>20</sub> ) Bases 3.8.4, Bases 3.8.5	The battery crosstie load limit is different for each station. The battery crosstie load limit for Byron is 63 amps and for Braidwood the limit is 100 amps.
	SR 4.8.2.1.2.a	SR 3.8.4.1	Based on different battery types, the specified battery terminal voltages are different for each station. The specified battery terminal voltage for Byron is 126 volts and for Braidwood the voltage is 130.5 volts.
	SR 4.8.2.1.2.d SR 4.8.2.1.2.e	SR 3.8.4.3, SR 3.8.4.4 Bases SR 3.8.4.3 Bases SR 3.8.4.4	The battery capacity tests are different for each station. Byron uses a performance discharge test and Braidwood uses a modified performance discharge test.
	Table 4.8-2	SR 3.8.6.6, SR 3.8.6.7 Bases SR 3.8.6.6 Bases SR 3.8.6.7	Float voltage and specific gravity parameters are different for each station.
	LCO 3.8.3.1 LCO 3.8.3.2	LCO 3.8.9, LCO 3.8.10 Bases 3.8.9	The listing of Distribution System buses are different for each station. Details in the Bases have been revised to reference two primary 480 V ESF buses at Byron and only one primary 480 V ESF bus at Braidwood.
3.9 REFUELING OPERATIONS	None	None	
4.0 DESIGN FEATURES	4.1.1 (DOC A <sub>2</sub> )	4.0	The site descriptions for each station are different.



**DIFFERENCES BETWEEN BYRON AND BRAIDWOOD**  
(page 4 of 4)

ITS SECTION	CTS LCO (Applicable DOC)	ITS LCO AND BASES (Applicable JFD)	DESCRIPTION OF DIFFERENCE
5.0 ADMINISTRATIVE CONTROLS	6.9.1.4 (DOC A <sub>8</sub> )	Not applicable	Deleted expired Byron time specific reporting requirement.
	6.9.1 (DOC A <sub>29</sub> )	Not applicable	Revised a Byron reference from "unit" to "facility" consistent with other CTS sections.
	4.0.5.b (DOC LA <sub>17</sub> )	Not applicable	Relocated Byron specific requirements associated with ASME Section XI testing requirements.
	SR 4.4.5.2, SR 4.4.5.4 SR 4.4.5.5.d	Spec 5.5.9.b, Spec 5.5.9.e Spec 5.6.9.d	The references to unit specific cycles associated with the steam generator inspection requirements are different for each station.
	SR 4.7.6, SR 4.7.7 (DOC A <sub>22</sub> and L <sub>3</sub> )	Spec 5.5.11	The specified train and bank flow rates for the Nonaccessible Area Exhaust Filter Plenum Ventilation System test and the specified wattage range for the Control Room Ventilation Filtration System heaters are different in the Ventilation Filter Testing Program each station.

ATTACHMENT 3

EXISTING AND FUTURE  
LICENSE AMENDMENT REQUESTS  
TO BE INCORPORATED INTO THE ITS

**EXISTING AND FUTURE LICENSE AMENDMENT REQUEST TO BE  
INCORPORATED INTO THE ITS**  
(page 1 of 2)

<b>LICENSE AMENDMENT REQUEST (Letter Title, Letter Date, Abbreviated Subject)</b>	<b>AFFECTED ITS LCO/SPECIFICATION</b>	<b>AFFECTED CTS PAGE(s)</b>	<b>ASSOCIATED DOC(s) AND JFD(s)</b>
Application for Amendment to Facility Operating Licenses: "Steam Generators," dated August 19, 1996.  (Steam Generator 3.0 volt criteria)	Specification 5.5.9 Specification 5.6.9	3/4 4-14 3/4 4-16 3/4 4-17 3/4 4-17a 3/4 4-17b 3/4 4-17c 3/4 4-17d	5.0 - A <sub>24</sub>
Application for Amendment to Facility Operating Licenses: "Non-Accessible Area Exhaust Filter Plenum Ventilation System," dated August 23, 1996.  (Nonaccessible Area Exhaust Filter Plenum Ventilation System)	LCO 3.7.12	3/4 7-17 (Braidwood) 3/4 7-19 (Byron)	3.7 - A <sub>44</sub>
Application for Amendment to Facility Operating Licenses-Reactivity Controls Systems, dated December 21, 1995; and Additional Information Regarding the Removal of Cycle-Specific Parameter Limits from Technical Specifications, dated October 24, 1996.  (Core Operating Limits Report)	Definitions	1-4	1.0 - A <sub>5</sub>
	LCO 3.1.4 LCO 3.1.5 LCO 3.1.6	3/4 1-14 3/4 1-15 3/4 1-20 3/4 1-21 3/4 1-22	3.1 - A <sub>15</sub>
	LCO 3.2.1 LCO 3.2.2 LCO 3.2.3	3/4 2-1 3/4 2-2 3/4 2-3 3/4 2-4 3/4 2-5 3/4 2-8	3.2 - A <sub>2</sub>
	6-22	Specification 5.6.5	5.0 - A <sub>30</sub>
Application for Amendment to Appendix A, Technical Specifications, for Facility Operating Licenses: "Containment Vessel Structural Integrity," dated November 4, 1996  (Tension Grease Removal).	None	3/4 6-8	5.0 - A <sub>35</sub>

**EXISTING AND FUTURE LICENSE AMENDMENT REQUEST TO BE  
INCORPORATED INTO THE ITS**  
(page 2 of 2)

<b>LICENSE AMENDMENT REQUEST (Letter Title, Letter Date, Abbreviated Subject)</b>	<b>AFFECTED ITS LCO/SPECIFICATION</b>	<b>AFFECTED CTS PAGE(s)</b>	<b>ASSOCIATED DOC(s) AND JFD(s)</b>
Application for Amendment to Appendix, Technical Specifications, for Facility Operating Licenses: Main Steam Safety Valves Technical Specification Change Request, dated April 29, 1996.  (MSSV's)	LCO 3.7.1	3/4 7-1 3/4 7-2 3/4 7-3	3.7 - A <sub>3</sub>
Application for Amendment to Appendix A, Technical Specifications 3.9.11, "Water Level-Storage Pool", 5.6.1.1, "Criticality," 6.9.1.10, "Criticality Analysis of Byron and Braidwood Station Fuel Storage Racks," dated November 5, 1996  (Boraflex) <sup>1</sup>	LCO 3.7.14 LCO 3.7.15 LCO 3.7.16	3/4 9-13	3.7 - A <sub>26</sub> 3.7 - P <sub>4</sub> (LCOs) 3.7 - P <sub>31</sub> (Bases)
	Specification 4.3.1	5-5	4.0 - A <sub>4</sub> 4.0 - P <sub>2</sub>
	None	6-23	5.0 - A <sub>16</sub>
Reactor Coolant System (RCS) Pressure and Temperature Limits Report (PTLR)  (PTLR) <sup>2</sup>	5.6.6	None	None

- As described in the associated Discussion of Changes (DOCs) and Justifications for Differences (JFDs), the ITS conversion does not include Specifications which may be impacted by the final resolution of the boraflex issue. A future request will provide these specifications.
- The initial PTLR will be provided for review and approval at a future date. Upon NRC review the dates which are blank in Specification 5.6.6 will be provided.

ATTACHMENT 4

PENDING AND PROPOSED  
ISTS CHANGE TRAVELERS



## PENDING AND PROPOSED ISTS CHANGE TRAVELERS

(page 1 of 11)

TSTF, WOG, EDITORIAL	GENERIC STATUS	ITS SPECs	ITS STATUS (1)	EXPLANATION OF INCORPORATION STATUS	LCO JFD(s)	BASES JFD(s)
TSTF-1 R1	Pending	3.0.5	F	NA	C <sub>3</sub>	C <sub>5</sub>
TSTF-2 R1	Pending	3.8.3	F	NA	C <sub>1</sub>	C <sub>5</sub>
TSTF-3 R1	Rejected	1.1	NI	Not incorporated based on being rejected.	--	--
		3.4.16	NI	Not incorporated based on being rejected.	--	--
		3.7.18	NI	Not incorporated based on being rejected.	--	--
TSTF-4 R1	Pending	1.1	P	Did not incorporate LTOP arming temperature in the PTLR definition and added PORV lift settings to 5.6.6.	C <sub>3</sub> /P <sub>5</sub>	--
		5.6.6	P	Did not incorporate LTOP arming temperature in the PTLR definition and added PORV lift settings to 5.6.6.	C <sub>9</sub>	--
TSTF-5 R1	Approved	2.0	FE	Editorial change made in B 2.1.1 to be consistent with B 2.1.2 change.	C <sub>1</sub>	C <sub>1</sub>
TSTF-6 R1	Approved	3.0.1	F	N/A	C <sub>1</sub>	--
TSTF-7 R1	Withdrawn	3.0.3	NI	Not incorporated based on being withdrawn.	--	--
TSTF-8 R1	Pending	3.8.1	P	Plant specific change makes changes to SR 3.5.1.12 and SR 3.5.1.13 not necessary.	C <sub>2</sub>	C <sub>2</sub> /P <sub>42</sub>
		B 3.0.1	F	N/A	--	C <sub>2</sub>
TSTF-9 R1	Approved	3.1	FE	Editorial change made to be consistent with the wording of other LCOs & Bases.	C <sub>1</sub>	C <sub>6</sub>
TSTF-10 R1	Pending	3.1	NI	Not incorporated based on inconsistencies with other LCOs.	--	--
TSTF-11 R1	Rejected	3.1.5	NI	Not incorporated based on being rejected.	--	--
TSTF-12 R1	Approved	3.0.7	F	N/A	C <sub>4</sub>	C <sub>3</sub>
		3.1	F	N/A	C <sub>5</sub>	C <sub>2</sub>

**PENDING AND PROPOSED ISTS CHANGE TRAVELERS**  
(page 2 of 11)

<b>TSTF, WOG, EDITORIAL</b>	<b>GENERIC STATUS</b>	<b>ITS SPECs</b>	<b>ITS STATUS (1)</b>	<b>EXPLANATION OF INCORPORATION STATUS</b>	<b>LCO JFD(s)</b>	<b>BASES JFD(s)</b>
TSTF-13 R1	Approved	3.1.3	FE	Editorial changes made in Bases consistent with other Bases discussions of Notes.	C <sub>2</sub>	C <sub>1</sub>
TSTF-14 R3	Pending	3.1.8	F	N/A	C <sub>7</sub>	C <sub>7</sub>
TSTF-15 R1	Pending	B 3.1.5	F	N/A	--	C <sub>3</sub>
TSTF-16 R1	Pending	3.8.9	P	Plant design dictates minor changes in Bases and not adopting LCO 3.8.9 Condition C changes.	C <sub>6</sub>	C <sub>7</sub>
TSTF-17 R1	Pending	3.6.2	FE	Revised Bases due to Byron/Braidwood 18 month fuel cycle.	C <sub>2</sub>	C <sub>2</sub>
TSTF-19	Pending	1.1	F	N/A	C <sub>2</sub>	--
		3.3	P	Changes to SR 3.3.1.12 were not made since the SR is not applicable to the station.	--	C <sub>2</sub> /P <sub>45</sub>
TSTF-20	Pending	3.9.7	F	N/A	C <sub>3</sub>	C <sub>1</sub>
TSTF-21 Proposed R1	Pending	3.9.6	F	N/A	C <sub>2</sub> /P <sub>4</sub>	C <sub>2</sub> /P <sub>24</sub>
TSTF-22	Pending	3.9.5 3.9.6	F	N/A	C <sub>2</sub>	--
TSTF-23 R1	Approved	B 3.9.3	P	Included requirement for visual indication of SRs; deleted requirement for audible count rate.	--	C <sub>5</sub> /P <sub>4</sub>
TSTF-24	Pending	3.2.3	F	N/A	C <sub>1</sub>	--
TSTF-25	Pending	3.2.4	F	N/A	C <sub>2</sub>	C <sub>2</sub>
TSTF-26	Approved	3.4.2	F	N/A	C <sub>1</sub>	C <sub>1</sub>
TSTF-27 R1	Pending	3.4.2	F	N/A	C <sub>2</sub>	C <sub>2</sub>
TSTF-28	Approved	3.4.16	FE	Editorial enhancements made to the Bases.	C <sub>3</sub>	C <sub>3</sub>

**PENDING AND PROPOSED ISTS CHANGE TRAVELERS**  
(page 3 of 11)

<b>TSTF, WOG, EDITORIAL</b>	<b>GENERIC STATUS</b>	<b>ITS SPECs</b>	<b>ITS STATUS (1)</b>	<b>EXPLANATION OF INCORPORATION STATUS</b>	<b>LCO JFD(s)</b>	<b>BASES JFD(s)</b>
TSTF-29	Rejected	3.7	NI	Incorporated plant specifically.	P <sub>2</sub>	P <sub>2</sub>
TSTF-30 R1	Pending	3.6.3	P	Incorporated as applicable (some Bases changes to Actions are not applicable to Byron and Braidwood design).	C <sub>4</sub>	C <sub>4</sub>
TSTF-36 R2	Pending	3.3.8	FE	Plant specific editorial enhancements made to the Bases due to the deletion of the second Note and to be consistent with other Bases Note discussions.	C <sub>7</sub>	C <sub>8</sub>
		3.8	F	N/A	C <sub>5</sub>	C <sub>3</sub>
		3.7.13	FE	Editorial enhancement to the Bases Note discussion.	C <sub>2</sub>	C <sub>3</sub>
TSTF-37	Pending	3.3.3	NI	Incorporated plant specifically.	--	--
		3.8.1	NI	Incorporated plant specifically.	--	--
		5.6	NI	Incorporated plant specifically.	--	--
TSTF-38	Approved	3.8	NI	Superseded by TSTF-115.	--	--
TSTF-39 R1	Pending	1.1	F	N/A	C <sub>7</sub>	--
TSTF-44 Proposed R1	Pending	3.6.3	P	Revision 0 included SG PORVS and MSSVs whose Containment Isolation Valve function is to close (Section 3.6) versus the System function which is to open (Section 3.7). Therefore, these valves were not included in the proposed Note in 3.6.3. Byron and Braidwood CTS do not include MSIV CIV requirements. Therefore, the MSIVs were not included in the Note.	C <sub>3</sub> /B <sub>6</sub>	C <sub>3</sub> /P <sub>5</sub>
		3.7.1 3.7.3 3.7.4	P	Byron and Braidwood CTS do not include MSIV CIV requirements. Therefore, the changes to the MSIV Bases are not included.	--	C <sub>4</sub> /B <sub>6</sub>
TSTF-45 R0	Pending	3.6	F	N/A	C <sub>5</sub>	C <sub>5</sub>

**PENDING AND PROPOSED ISTS CHANGE TRAVELERS**  
(page 4 of 11)

<b>TSTF, WOG, EDITORIAL</b>	<b>GENERIC STATUS</b>	<b>ITS SPECs</b>	<b>ITS STATUS (1)</b>	<b>EXPLANATION OF INCORPORATION STATUS</b>	<b>LCO JFD(s)</b>	<b>BASES JFD(s)</b>
TSTF-46 R0	Pending	3.6	F	N/A	C <sub>6</sub>	C <sub>6</sub>
TSTF-51	Pending	Various	NI	Not incorporated based on current licensing bases provide adequate allowance for containment penetrations during refueling.	--	--
TSTF-52	Pending	1.1	FE	Added as editorial.	C <sub>11</sub>	--
		B 3.0	F	N/A	--	C <sub>6</sub> /B <sub>4</sub>
		3.6	P	Editorially revised 3.6 Bases (= signs) and incorporated as pertinent to Byron and Braidwood design.	C <sub>1</sub>	C <sub>1</sub>
		5.5	FE	Editorially revised Section 5.0 (= signs and acceptance criteria). Incorporated plant specific CTS air lock acceptance criteria.	C <sub>5</sub> /P <sub>12</sub>	--
TSTF-54 R0	Pending	B 3.4.13	P	Revised to replace the term "identified" with the term "unidentified."	--	C <sub>4</sub>
TSTF-60	Approved	3.4.15	NI	Contradicts Condition D for entering LCO 3.0.3.	--	--
TSTF-61	Approved	3.4	F	N/A	C <sub>11</sub>	--
TSTF-64	Pending	1.1	F	N/A	C <sub>8</sub>	--
TSTF-65	Pending	2.0	NI	Section 2.0 change superseded by TSTF-5. Some TSTF changes were superseded by plant specific nomenclature/preferences and some were not applicable depending upon which bracketed option was chosen.	--	--
		5.0	P	Some TSTF changes were superseded by plant specific nomenclature/preferences and some were not applicable depending upon which bracketed option was chosen.	C <sub>1</sub>	--
TSTF-68 R0	Pending	3.9.4	NI	Requires plant specific calculation to support.	--	--

**PENDING AND PROPOSED ISTS CHANGE TRAVELERS**  
(page 5 of 11)

<b>TSTF, WOG, EDITORIAL</b>	<b>GENERIC STATUS</b>	<b>ITS SPECs</b>	<b>ITS STATUS (1)</b>	<b>EXPLANATION OF INCORPORATION STATUS</b>	<b>LCO JFD(s)</b>	<b>BASES JFD(s)</b>
TSTF-70	Approved	3.7.16	NI	See Section 3.7 DOC A <sub>26</sub> .	--	--
TSTF-71	Pending	B 3.0	NI	A plant specific example may be better placed in the Bases.	--	--
TSTF-86	Rejected	5.2	NI	Not incorporated based on being rejected.	--	--
TSTF-87 R1	Pending	3.4.5 3.4.9	P	Specified the Required Action as "Initiate action to place..." rather than "Place..." on an "Immediate" frequency.	C <sub>10</sub> /P <sub>37</sub>	C <sub>10</sub>
TSTF-88	Pending	T 1.1-1	F	N/A	C <sub>9</sub>	--
TSTF-89	Approved	3.1.7	FE	Plant specific editorial enhancement.	C <sub>8</sub>	C <sub>8</sub>
TSTF-90	Pending	3.5.3	P	Retained Operating SRs consistent with CTS.	C <sub>1</sub>	C <sub>1</sub>
TSTF-91	Pending	3.3.5	P	Bracketed Trip Setpoints moved to Bases.	C <sub>3</sub>	C <sub>3</sub>
TSTF-92 R0	Pending	3.9.4	NI	The proposed change does not fully address the problem. This issue is covered per the word "required" in the SR. The issue is related to LCO 3.3.6 and LCO 3.9.4.	--	--
TSTF-93	Pending	3.4.9	FE	Plant specific editorial enhancements made to the Bases.	C <sub>5</sub>	C <sub>5</sub>
TSTF-94	Pending	3.4.9	NI	TSTF only provides brackets for wording that was incorporated per CTS	--	--
TSTF-95	Approved	3.2.1 3.2.2	F	N/A	C <sub>3</sub>	C <sub>3</sub>
TSTF-96	Approved	3.9.3	F	N/A	C <sub>5</sub>	C <sub>4</sub>
TSTF-97	Approved	3.2.1	F	N/A	C <sub>4</sub>	C <sub>4</sub>
TSTF-98 R1	Approved	3.2.1	FE	Plant specific editorial enhancement made to the LCO to reference the COLR.	C <sub>5</sub> /P <sub>3</sub>	C <sub>5</sub>
TSTF-99	Approved	3.2.1	F	N/A	C <sub>6</sub>	C <sub>6</sub>



**PENDING AND PROPOSED ISTS CHANGE TRAVELERS**  
(page 6 of 11)

<b>TSTF, WOG, EDITORIAL</b>	<b>GENERIC STATUS</b>	<b>ITS SPECs</b>	<b>ITS STATUS (1)</b>	<b>EXPLANATION OF INCORPORATION STATUS</b>	<b>LCO JFD(s)</b>	<b>BASES JFD(s)</b>
TSTF-100	Approved	3.7.4	F	N/A	C <sub>7</sub>	--
TSTF-101	Approved	3.7.5	F	N/A	C <sub>4</sub>	C <sub>5</sub>
TSTF-102 R0	Pending	3.7.2 3.7.3	P	LCO JFD P <sub>20</sub> combines the Actions which makes the proposed changes to LCO 3.7.3 Conditions B and C unnecessary.	C <sub>6</sub> /P <sub>20</sub>	C <sub>7</sub> /P <sub>36</sub>
TSTF-103	Pending	3.0.4	F	N/A	C <sub>5</sub> /P <sub>1</sub>	C <sub>7</sub>
TSTF-104	Pending	3.0.4	F	N/A	C <sub>6</sub>	C <sub>8</sub>
TSTF-105	Pending	3.4.1	FE	Editorial enhancement made to proposed insert.	C <sub>6</sub>	C <sub>5</sub>
TSTF-106 R1	Approved	5.0	NI	Not incorporated due to inconsistencies with 3.8 Bases.	----	--
TSTF-107	Pending	3.1	FE	Editorial change made to proposed insert.	C <sub>9</sub>	C <sub>9</sub>
TSTF-108	Pending	3.1.8	F	N/A	C <sub>11</sub>	C <sub>11</sub>
		3.4.19	NI	LCO 3.4.19 "RCS Loops-Test Exceptions" is for Near Term Operating License plants	P <sub>30</sub>	P <sub>35</sub>
TSTF-109	Approved	3.2.4	F	N/A	C <sub>7</sub>	C <sub>7</sub>
TSTF-110 R1	Pending	3.1.4 3.1.6	F	N/A	C <sub>10</sub>	C <sub>10</sub>
		3.2.3 3.2.4	F	N/A	C <sub>9</sub>	C <sub>9</sub>
TSTF-111	Pending	B 3.3.1 B 3.3.2	F	N/A	--	C <sub>7</sub>
TSTF-112	Pending	3.2.3	F	N/A	C <sub>8</sub>	C <sub>8</sub>
TSTF-113 R1	Pending	3.4.11	F	N/A	C <sub>12</sub>	C <sub>13</sub>
TSTF-114	Pending	3.4.7	FE	Editorial enhancements made to the Bases.	--	C <sub>11</sub> /P <sub>3</sub>

**PENDING AND PROPOSED ISTS CHANGE TRAVELERS**  
(page 7 of 11)

<b>TSTF, WOG, EDITORIAL</b>	<b>GENERIC STATUS</b>	<b>ITS SPECs</b>	<b>ITS STATUS (1)</b>	<b>EXPLANATION OF INCORPORATION STATUS</b>	<b>LCO JFD(s)</b>	<b>BASES JFD(s)</b>
TSTF-115	Pending	3.8	P	Some 3.8.4 Bases modifications are not applicable due to plant specific design.	C <sub>7</sub> /C <sub>8</sub>	C <sub>4</sub> /C <sub>6</sub> /P <sub>37</sub> P <sub>43</sub>
		5.0	FE	Some 3.8.4 Bases modifications are not applicable due to plant specific design.	C <sub>6</sub>	--
TSTF-116	Pending	3.4.13 3.4.15	FE	Editorial enhancement made to clarify the intent of steady state operation.	C <sub>8</sub> /P <sub>23</sub>	C <sub>8</sub> /P <sub>23</sub>
TSTF-117 (WOG-57)	Pending	3.5.1	P	Plant Specific deletion of details in SR 3.5.1.6.	C <sub>4</sub> /P <sub>8</sub>	C <sub>4</sub> /P <sub>4</sub>
TSTF-135 (WOG-58)	Pending	3.3	P	Various plant specific changes were made (marked against clean copy of WOG-58).	C <sub>1</sub>	C <sub>1</sub>
TSTF-136 (WOG-59)	Pending	3.0.7	F	N/A	C <sub>7</sub>	C <sub>9</sub>
		3.1	FE	Changes made consistent with the incorporation of TSTF-12. Revision 1 changes.	C <sub>4</sub>	C <sub>4</sub>
TSTF-151 (WOG-61)	Pending	B 3.4.11	F	N/A	--	C <sub>14</sub>
TSTF-152 (WOG-65)	Pending	5.6	F	N/A	C <sub>4</sub>	--
TSTF-153 (WOG-63)	Pending	3.4.5	F	N/A	C <sub>14</sub>	C <sub>18</sub>
		3.5.2	P	LCO Note 2 was deleted plant specifically.	C <sub>3</sub> /B <sub>5</sub>	C <sub>3</sub> /P <sub>11</sub> /P <sub>12</sub>
BWR-06, C.3, R1 (2)	Pending	1.0	F	Submitted as NRC editorial.	C <sub>5</sub> /C <sub>6</sub>	--
BWR-11, C.10, R1 (2)	Pending	2.0	F	Submitted as NRC editorial.	--	C <sub>2</sub>

## PENDING AND PROPOSED ISTS CHANGE TRAVELERS

(page 8 of 11)

TSTF, WOG, EDITORIAL	GENERIC STATUS	ITS SPECs	ITS STATUS (1)	EXPLANATION OF INCORPORATION STATUS	LCO JFD(s)	BASES JFD(s)
WOG-21	Withdrawn	3.4.1	F	This change was pending at time of development; but later withdrawn.	C <sub>7</sub>	C <sub>7</sub>
WOG-51 R1	Pending	3.4	P	Plant specific changes were made consistent with Current Licensing Basis.	C <sub>9</sub> /P <sub>11</sub> /P <sub>21</sub>	C <sub>9</sub> /P <sub>11</sub> /P <sub>19</sub> P <sub>21</sub>
WOG-56 (BWOG-13)	Withdrawn	5.0	P	Proposed changes to Specification 5.2.2.e are not required due to which bracketed option was chosen.	C <sub>2</sub> /B <sub>3</sub>	--
WOG-60	Pending	3.4	P	Editorial enhancements made to the Bases.	C <sub>13</sub>	C <sub>15</sub>
WOG-62	Pending	3.8	FE	Editorial enhancements made to the Bases.	C <sub>9</sub> /P <sub>15</sub>	C <sub>1</sub> /P <sub>37</sub>
WOG-64	Pending	3.7	FE	Editorial change to add separate condition entry for Condition C consistent with Condition D.	C <sub>8</sub>	C <sub>8</sub>
WOG-66	Pending	3.3	P	Modified Table 3.3.6-1 footnote. Editorial enhancements made to the Bases.	C <sub>4</sub>	C <sub>4</sub>
WOG-67	Pending		NI	See TSTF-4 R1	--	--
WOG-68	Pending	3.4	F	N/A	--	C <sub>17</sub>
WOG-69	Pending	3.8	P	Proposed changes to SR 3.8.1.2 not needed because, SR 3.8.1.2 and SR 3.8.1.7 are applicable to Byron and Braidwood.	C <sub>11</sub> /P <sub>10</sub>	--
WOG-70	Pending	3.8	P	Proposed changes to SR 3.8.1.2 not needed because, SR 3.8.1.2 and SR 3.8.1.7 are applicable to Byron and Braidwood. Change to SR 3.8.1.2 to delete standby is editorially moved to WOG-71.	C <sub>12</sub>	C <sub>10</sub>
WOG-71	Pending	3.8	FE	Proposed change to SR 3.8.1.2 to delete standby is editorially added to WOG-71	C <sub>13</sub>	--
WOG-72	Pending	5.0	FE	Editorial enhancement made to description.	C <sub>8</sub>	--

## PENDING AND PROPOSED ISTS CHANGE TRAVELERS

(page 9 of 11)

TSTF, WOG, EDITORIAL	GENERIC STATUS	ITS SPECs	ITS STATUS (1)	EXPLANATION OF INCORPORATION STATUS	LCO JFD(s)	BASES JFD(s)
WOG-73	Pending	3.1	FE	Editorial enhancement made to the LCO to replace "position indicators" with "DRPI"	C <sub>13</sub> /P <sub>10</sub>	C <sub>13</sub>
WOG-74	Pending	1.0	FE	Editorial enhancement made to correct grammar.	C <sub>10</sub>	--
WOG-75	Pending	3.2	P	Incorporated plant specific corrections in the LCO and plant specific editorial changes in the Bases. The changes explicitly state the power bands to which they apply.	C <sub>10</sub>	C <sub>11</sub>
WOG-76	Pending	3.9	NI	Requires a plant specific calculation to implement.	--	--
WOG-77	Pending	B 3.0	F	N/A	--	C <sub>10</sub>
WOG-78	Pending	3.0	F	N/A	C <sub>8</sub>	--
WOG-79	Pending	5.0	NI	Incorporation of inadvertent would be less restrictive than CTS and was not justifiable.	--	--
WOG-80	Pending	3.3	P	Changes made consistent with incorporation of TSTF-135 changes.	C <sub>5</sub>	C <sub>5</sub>
WOG-81	Pending	3.1	F	N/A	C <sub>12</sub>	--
		3.3	F	N/A	C <sub>6</sub>	--
		3.4	F	N/A	C <sub>15</sub>	--
		3.5	F	N/A	C <sub>2</sub>	--
		3.7	F	N/A	C <sub>9</sub>	--
		3.8	F	N/A	C <sub>10</sub>	--
		3.9	F	N/A	C <sub>4</sub>	--
WOG-82	Pending	3.3	NI	Not incorporated based on the lack of technical justification.	--	--

**PENDING AND PROPOSED ISTS CHANGE TRAVELERS**  
(page 10 of 11)

<b>TSTF, WOG, EDITORIAL</b>	<b>GENERIC STATUS</b>	<b>ITS SPECs</b>	<b>ITS STATUS (1)</b>	<b>EXPLANATION OF INCORPORATION STATUS</b>	<b>LCO JFD(s)</b>	<b>BASES JFD(s)</b>
WOG-83	Pending	3.7	P	Bases changes to SR 3.7.1.1 are not applicable to Byron and Braidwood (See LCO JFD P <sub>58</sub> ).	C <sub>3</sub> /B <sub>22</sub> /B <sub>2</sub>	C <sub>2</sub> /B <sub>21</sub>
Editorial-1	Pending	B 3.1	F	N/A	--	C <sub>12</sub>
		B 3.2	F	N/A	--	C <sub>1</sub>
		B 3.3	F	N/A	--	C <sub>6</sub>
		B 3.4	F	N/A	--	C <sub>15</sub>
		B 3.5	F	N/A	--	C <sub>2</sub>
		B 3.6	F	N/A	--	C <sub>7</sub>
		B 3.7	F	N/A	--	C <sub>9</sub>
		B 3.8	F	N/A	--	C <sub>9</sub>
		B 3.9	F	N/A	--	C <sub>3</sub>
Editorial-2	Pending	B 3.1.1	P	Superseded by plant specific changes.	--	P <sub>4</sub> /P <sub>8</sub>
Editorial-3	Pending	3.2.4	F	N/A	C <sub>11</sub>	C <sub>10</sub>
Editorial-4	Pending	3.3	NI	Technically incorrect. Incorporated proposed Editorial-4 R1.	B <sub>2</sub>	--
Editorial-5	Pending	3.3	NI	Superseded by plant specific changes.	P <sub>31</sub>	P <sub>31</sub> /P <sub>54</sub>



**PENDING AND PROPOSED ISTS CHANGE TRAVELERS**  
(page 11 of 11)

<b>TSTF, WOG, EDITORIAL</b>	<b>GENERIC STATUS</b>	<b>ITS SPECs</b>	<b>ITS STATUS (1)</b>	<b>EXPLANATION OF INCORPORATION STATUS</b>	<b>LCO JFD(s)</b>	<b>BASES JFD(s)</b>
Editorial-6	Pending	3.8	P	Changes to 3.8.6 Bases superseded by TSTF-115.	C <sub>4</sub>	C <sub>8</sub>
Editorial-7	Pending	5.0	F	N/A	C <sub>7</sub>	--
Editorial-8	Pending	3.5	NI	Superseded by plant specific change.	B <sub>2</sub>	--

---

Table Notations:

- (1) F - Fully Incorporated  
 FE - Fully incorporated with editorial enhancements  
 P - Partially incorporated  
 NI - Not incorporated

- (2) The TSTF numbers refer to NUREG-1431, Revision 0 generic changes which were not correctly incorporated into NUREG-1431, Revision 1.



ATTACHMENT 5

"BEYOND SCOPE" CHANGES

**"BEYOND SCOPE" CHANGES**

(page 1 of 2)

No.	ITS	CTS	EXPLANATION OF CHANGE	CTS DOC	LCO JFD
1	LCO 3.1.5 Condition B	LCO 3.1.3.5	Changed the Required Action for not meeting LCO to agree with the Applicability.	M <sub>6</sub>	P <sub>3</sub>
2	LCO 3.1.7	LCO 3.1.3.2.a	Deleted Action to "immediately" verify the position of the non-indicating rod after any motion which exceeds 24 steps, based on the existing Action to verify position for non-indicating rods every 8 hours.	A <sub>13</sub>	P <sub>11</sub>
3	LCO 3.3.2 Condition M	Table 3.3-3 Action 15a	Changed the Completion Time for declaring associated AF pump inoperable from immediately to 48 hours.	L <sub>7</sub>	P <sub>16</sub>
4	SR 3.3.9.7	SR 4.1.2.7.a Table 4.3-1	The SRM Surveillance is revised to allow the Surveillance to be performed after entering the Mode of Applicability. This is consistent with allowances provided in SR 3.3.1.7 and SR 3.3.1.8.	L <sub>11</sub>	P <sub>43</sub>
5	SR 3.4.1.4 Note	SR 4.2.3.5	Changed the Surveillance Frequency exception from 24 hours to 7 days (NUREG), and from unspecified to 7 days (CTS).	M <sub>15</sub>	P <sub>3</sub>
6	LCO 3.4.3	LCO 3.4.9.1	Revised P/T Limits Action for P/T Limits not within Specification to place the unit in MODE 5 rather than in MODE 5 with RCS pressure < 500 psig.	L <sub>28</sub>	P <sub>44</sub>
7	LCO 3.4.18	LCO 3.4.1.5.2	Revised the standard by which the isolated loop is compared to allow opening of the isolation valves. This change requires the isolated loop boron concentration to be greater than or equal to the "required shutdown margin boron concentration," rather than the operating loop boron concentration.	L <sub>18</sub>	P <sub>31</sub>
8	SR 3.4.18.2	SR 4.4.1.5.2.2	Changed Surveillance Frequency from 2 hours to 4 hours for verifying boron concentration prior to opening an loop stop isolation valve.	L <sub>14</sub>	P <sub>4</sub>
9	LCO 3.5.5	LCO 3.4.6.2.e	Changed the limits for seal injection flow. CTS and NUREG specify a unique value. Byron and Braidwood specify a range of values in Figure 3.5.5-1.	L <sub>4</sub>	P <sub>9</sub>

**"BEYOND SCOPE" CHANGES**  
(page 2 of 2)

No.	ITS	CTS	EXPLANATION OF CHANGE	CTS DOC	LCO JFD
10	LCO 3.6.3	LCO 3.6.1.7	Added clarification that deactivated remote manual valves can satisfy the Required Action for an inoperable containment isolation valve to be isolated.	A <sub>25</sub>	P <sub>2</sub>
11	LCO 3.8.1	LCO 3.8.1.1	Revised Required Action for loss of 2 offsite circuits to one bus and loss of the associated DG from enter LCO 3.0.3 to restore in 8 hours.	L <sub>28</sub>	P <sub>22</sub>
12	SR 3.8.1.8	SR 4.8.1.1.1.b	Deleted the "during shutdown" aspect of the SRs. (Deleted ITS Note that indicates "This Surveillance shall not be performed in MODE 1 or 2".)	L <sub>26</sub>	P <sub>21</sub>
	SR 3.8.1.12 SR 3.8.1.13	SR 4.8.1.1.2.f.5) SR 4.8.1.1.2.f.6)		L <sub>29</sub>	P <sub>23</sub>
13	SR 3.8.1.10	SR 4.8.1.1.2.f.3)	Added Note such that momentary transients above the DG full load reject voltage limit do not invalidate the test	L <sub>24</sub>	P <sub>27</sub>
14	LCO 3.9.4	LCO 3.9.4	Added Surveillance Requirement to verify purge valve isolation times.	M <sub>8</sub>	P <sub>3</sub>
15	Specification 5.5.11	LCO 3.7.6 LCO 3.7.7 LCO 3.9.12	Added clarification that the Ventilation Filter Testing Program tests may be in "general conformance with" Regulatory Guide 1.52.	L <sub>7</sub>	P <sub>4</sub>
16	Specification 5.5.16	SR 4.6.1.2	Added exception to NEI 94-01 in CTS Section 3.6 to allow the elapsed time between consecutive satisfactory Type A tests to be $\geq$ 18 months rather than $\geq$ 24 months.	L <sub>4</sub>	C <sub>5</sub>
17	Specification 5.7	Specification 6.12	Revised High Radiation Area Specification to reflect the most current 10 CFR Part 20.	L <sub>2</sub>	P <sub>11</sub>

ATTACHMENT 6

"BEYOND SCOPE BRACKETED" CHANGES

## BEYOND SCOPE BRACKETED CHANGES

(page 1 of 1)

No.	ITS	CTS	EXPLANATION OF CHANGE	DOC
1	LCO 3.2.1	LCO 3.2.2	Changed current licensing basis methodology used to determine Power Distribution Limit (Heat Flux Hot Channel Factor).	L <sub>9</sub>
2	SR 3.4.5.2 SR 3.4.6.2 SR 3.4.7.2	SR 4.4.1.3.2	Changed low low SG Water Level values consistent with Rx Trip setpoints.	L <sub>5</sub>
3	LCO 3.7.12	LCO 3.7.7	(Braidwood Only) Modified SRs such that HVAC bank flow rates need only be measured after structural housing maintenance.	L <sub>27</sub>
	Specification 5.5			L <sub>3</sub>
4	SR 3.8.1.3	SR 4.8.1.1.2.a.5)	Change DG loading values; in conformance with Regulatory Guide 1.9.	L <sub>25</sub>
	SR 3.8.1.10	SR 4.8.1.1.2.f.3)		L <sub>25</sub>
	SR 3.8.1.14	SR 4.8.1.1.2.f.7) & Footnote *		L <sub>25</sub> /L <sub>27</sub>
	SR 3.8.1.15	SR 4.8.1.1.2.f.7) & Footnote **		L <sub>27</sub>
5	LCO 3.8.1 SRs	SR 4.8.1.1.2 SRs	Revised acceptable voltages from 4160 V $\pm$ 420 V to 3950 V to 4580 V.	M <sub>2</sub>

ATTACHMENT 7

CTS REQUIREMENTS RELOCATED  
TO OTHER LICENSEE CONTROLLED DOCUMENTS



## CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS

(page 1 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
ITS SECTION: 2.0 SAFETY LIMITS			
LA <sub>1</sub>	The requirement to notify the NRC within 1 hour following violation of a Safety Limit.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>2</sub>	The requirement to notify the Vice President PWR Operations and the Offsite Review and Investigation Function within 24 hours following violation of a Safety Limit.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>3</sub>	The requirement to prepare, review, and submit a Safety Limit Violation Report within 14 days to the NRC, appropriate management personnel, and plant safety committees following a Safety Limit violation.	UFSAR Chapter 17	10CFR50.54(a)
LA <sub>4</sub>	The requirement to obtain authorization by the Commission prior to critical operation of the unit following a Safety Limit violation.	TRM <sup>(1)</sup>	10CFR50.59
ITS SECTION: 3.0 LCO AND SR APPLICABILITY			
LA <sub>1</sub>	The Applicability requirements providing guidance for dual unit operation to clarify each LCO applicability or surveillance activity.	TRM <sup>(1)</sup>	10CFR50.59
ITS SECTION: 3.1 REACTIVITY CONTROL SYSTEMS			
R <sub>1</sub>	The requirement associated with establishing the controls for shutdown margin in MODES 1 and 2 with $k_{eff} \geq 1.0$ .	TRM <sup>(1)</sup>	10CFR50.59
R <sub>2</sub>	The requirement associated with establishing the flow path controls for the boration subsystem of the Chemical and Volume Control System in MODES 4, 5, and 6.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>3</sub>	The requirement associated with establishing the flow path controls for the boration subsystem of the Chemical and Volume Control System in MODES 1, 2, and 3.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>4</sub>	The requirement associated with establishing the charging pump controls for the boration subsystem of the Chemical and Volume Control System in MODES 4, 5, and 6.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>5</sub>	The requirement associated with establishing the charging pump controls for the boration subsystem of the Chemical and Volume Control System in MODES 1, 2, and 3.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>6</sub>	The requirement to establish the controls for the control rod position indicating system in MODES 3, 4, and 5 when the reactor trip breakers are closed and during performance of rod drop time measurements.	TRM <sup>(1)</sup>	10CFR50.59

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 2 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.1 REACTIVITY CONTROL SYSTEMS (continued)</b>			
LA <sub>1</sub>	The specific values associated with Shutdown Margin supporting reactivity control.	COLR <sup>(1)</sup>	10CFR50.59
LA <sub>2</sub>	Details which specify the required boration flow rate and boric acid solution concentration associated with the method of restoring Shutdown Margin compliance.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>3</sub>	Procedural details describing the reactivity factors taken into consideration when performing a reactivity balance calculation.	Bases	Bases Control Program
LA <sub>4</sub>	Details associated with the administrative control bank withdrawal limits which are established when Moderator Temperature Coefficient is more positive than the beginning of life Moderator Temperature Coefficient limit.	Bases	Bases Control Program
LA <sub>5</sub>	The requirement to submit a Special Report within 10 days to the NRC in the event Moderator Temperature Coefficient is more positive than the beginning of life limit specified in the Core Operating Limits Report.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>6</sub>	Descriptive information associated with two possible examples of what could render a control rod immovable.	Bases	Bases Control Program
LA <sub>7</sub>	Descriptive information associated with an alternative approach for the restoration of a misaligned rod.	Bases	Bases Control Program
LA <sub>8</sub>	Details associated with the list of accident analyses to be addressed in the re-evaluation in the event of an inoperable but trippable rod have been relocated.	UFSAR	10CFR50.59
LA <sub>9</sub>	The requirement associated with a more frequent performance of the rod position or control bank surveillance when the rod position deviation alarm is inoperable.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>10</sub>	The requirement associated with performing a rod drop test for affected rods following any maintenance on or modification to the Control Rod Drive System.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>11</sub>	Descriptive information associated with the requirements which must be met for the control rod position indicators to be Operable.	Bases	Bases Control Program
LA <sub>12</sub>	The requirement associated with verifying that each shutdown bank is within the insertion limit specified in the Core Operating Limits Report within 15 minutes prior to withdrawal of any control bank rods during an approach to criticality.	TRM <sup>(1)</sup>	10CFR50.59

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 3 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.1 REACTIVITY CONTROL SYSTEMS (continued)</b>			
LA <sub>13</sub>	Details associated with the method of establishing compliance and restoring the control banks to within limits by reducing thermal power consistent with rod position.	Bases	Bases Control Program
LA <sub>14</sub>	Details associated with the method for verifying the Heat Flux Hot Channel Factor and the Nuclear Enthalpy Rise Hot Channel Factor are within limits.	Bases	Bases Control Program
LA <sub>15</sub>	The requirement associated with establishing the borated water source controls for the Boric Acid Storage System and the Refueling Water Storage Tank.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>16</sub>	The requirement associated with performing additional Shutdown Margin verifications after detection of an inoperable control rod(s) in Mode 2 with $k_{eff} < 1.0$ and Modes 3, 4, and 5.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>17</sub>	Details associated with the initial conditions for verifying core reactivity.	TRM <sup>(1)</sup>	10CFR50.59
<b>ITS SECTION: 3.2 POWER DISTRIBUTION LIMITS</b>			
LA <sub>1</sub>	Details associated with the method for confirming the normalized symmetric power distribution during the Quadrant Power Tilt Ratio surveillances.	Bases	Bases Control Program
LA <sub>2</sub>	Details associated with the Heat Flux Hot Channel Factor limit.	Bases/COLR <sup>(1)</sup>	Bases Control Program/ 10CFR50.59
LA <sub>3</sub>	Details associated with the Nuclear Enthalpy Rise Hot Channel Factor limit.	Bases/COLR <sup>(1)</sup>	Bases Control Program/ 10CFR50.59
LA <sub>4</sub>	Details associated with the optional method in which the target flux difference may be determined during an Axial Flux Difference surveillance.	Bases	Bases Control Program
LA <sub>5</sub>	The Heat Flux Hot Channel Factor and Nuclear Enthalpy Rise Hot Channel Factor requirement to identify and correct the cause of the out of limit condition prior to increasing thermal power.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>6</sub>	The requirement to provide conditional surveillances whenever the Axial Flux Difference monitor alarm and Quadrant Power Tilt Ratio monitor alarm are inoperable.	TRM <sup>(1)</sup>	10CFR50.59

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 4 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.2 POWER DISTRIBUTION LIMITS (continued)</b>			
LA <sub>7</sub>	Details associated with the method for determining the target flux difference during surveillance.	Bases	Bases Control Program
LA <sub>8</sub>	Details associated with demonstrating that the Heat Flux Hot Channel Factor is returned to within limits after corrective actions have been taken through incore flux mapping.	Bases	Bases Control Program
<b>ITS SECTION: 3.3 INSTRUMENTATION</b>			
R <sub>1</sub>	The requirement to ensure the Operability of the Movable Incore Detector Instrumentation for use in monitoring the flux distribution within the core.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>2</sub>	The requirement to ensure the Operability of seismic instrumentation used to record data for use in evaluating the effect of a seismic event after the occurrence of such an event.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>3</sub>	The requirement to ensure the Operability of meteorological instrumentation used to record meteorological data for use in evaluating the effect of an accidental radioactive release from the plant.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>4</sub>	The requirement to ensure the Operability of loose part detection instrumentation used to detect loose parts in the Reactor Coolant System which could cause damage to components in the Reactor Coolant System.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>5</sub>	The requirement to ensure the Operability of high-energy line break isolation sensors used to detect and initiate protective actions in the event of a line break.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>6</sub>	The requirement to ensure the Operability of turbine overspeed protection instrumentation used to trip the turbine to prevent the generation of potentially damaging missiles from the turbine, in the event of a loss of the turbine speed control system, or a transient.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>7</sub>	The requirement to ensure the Operability of the explosive gas monitoring instrumentation for maintaining the concentration of potentially explosive gas mixtures contained in the Waste Gas Holdup System below the flammability limit of hydrogen and oxygen.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>1</sub>	Details of the valves to be closed and the method of isolating the unborated water source flow paths in the event of a Boron Dilution Prevention System inoperability.	Bases	Bases Control Program

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 5 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.3 INSTRUMENTATION (continued)</b>			
LA <sub>2</sub>	Details of Source Range Monitor Operability supporting the Operability of the Boron Dilution Protection System.	Bases	Bases Control Program
LA <sub>3</sub>	Allowable Values for the Loss of Power ESF Bus Undervoltage and Degraded Grid Voltage Functions.	Bases	Bases Control Program
LA <sub>4</sub>	Details associated with the Reactor Trip System and Engineered Safety Feature Actuation System interlock function requirements and the descriptive information for each interlock function.	Bases/TRM <sup>(1)</sup>	Bases Control Program/ 10CFR50.59
LA <sub>5</sub>	Design details associated with the Reactor Trip System and Engineered Safety Feature Actuation System functions.	Bases/UFSAR	Bases Control Program/ 10CFR50.59
LA <sub>7</sub>	Details associated with the Reactor Trip System, Engineered Safety Feature Actuation System, and Containment Ventilation Isolation Instrumentation trip setpoints.	ODCM/TRM	ITS ODCM Program/ 10CFR50.59
LA <sub>8</sub>	Details associated with the Reactor Trip System and Engineered Safety Feature Actuation System instrumentation allowable values.	Bases	Bases Control Program
LA <sub>9</sub>	Details associated with the Boron Dilution Protection System test acceptance criteria for an actuation test.	Bases	Bases Control Program
LA <sub>11</sub>	Details associated with performing Channel Calibrations on the Intermediate Range Monitor and Source Range Monitor instruments.	Bases	Bases Control Program
LA <sub>12</sub>	The requirement to perform a Trip Actuating Device Operational Test following maintenance or adjustment of the Reactor Trip Breakers.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>13</sub>	Details associated with the description of the Engineered Safety Feature Actuation System instrumentation design.	Bases	Bases Control Program
LA <sub>14</sub>	The requirement to maintain the Engineered Safety Feature Actuation System function for individual manual isolation of the main steam isolation valves.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>15</sub>	The requirement to maintain the Engineered Safety Feature Actuation System function for manual initiation of the auxiliary feedwater pumps.	TRM <sup>(1)</sup>	10CFR50.59



**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 6 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
ITS SECTION: 3.3 INSTRUMENTATION (continued)			
LA <sub>16</sub>	Details of performing Channel Calibrations on the Steam Line Pressure - Low and Steam Line Pressure - Negative Rate - High Engineered Safety Feature Actuation System Functions.	Bases	Bases Control Program
LA <sub>17</sub>	Instrument numbers associated with the Fuel Building Isolation, Containment Isolation, and Main Control Room Isolation Radiation Monitoring Functions.	Bases	Bases Control Program
LA <sub>18</sub>	The requirement to submit a Special Report within 30 days to the NRC following the inability to restore the Main Control Room Isolation Radiation Monitoring Function to an Operable status.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>19</sub>	Details and requirements associated with for the operation of the crane with loads over the fuel storage pool.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>20</sub>	Design attributes associated with the Fuel Handling Building Exhaust Filter Plenum Ventilation System.	Bases	Bases Control Program
LA <sub>21</sub>	The requirement to provide a portable continuous monitor with the same alarm setpoint in the fuel pool area whenever both fuel handling building isolation channels are inoperable.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>22</sub>	Design attributes associated with the Remote Shutdown System.	Bases	Bases Control Program
LA <sub>23</sub>	The listing of Remote Shutdown System Functions.	Bases	Bases Control Program
LA <sub>24</sub>	The requirements associated with the Accident Monitoring System Functions which are not classified as Regulatory Guide 1.97, Type A and Category I variables.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>25</sub>	Details associated with the design of Engineered Safety Feature Actuation System Functions.	Bases	Bases Control Program
LA <sub>26</sub>	Details associated with the method of performing Channel Calibrations on the Post Accident Monitoring - Containment High Range Area Radiation instruments.	Bases	Bases Control Program



## CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS

(page 7 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
ITS SECTION: 3.3 INSTRUMENTATION (continued)			
LA <sub>27</sub>	The requirement to perform an analog Channel Operational Test every 92 days on the hydrogen monitor and a requirement to check that the hydrogen monitor is in a standby mode.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>28</sub>	Details associated with the method of performing Channel Calibrations on the Hydrogen Monitor instrumentation.	Bases	Bases Control Program
LA <sub>30</sub>	Design attributes associated with the Remote Shutdown System.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>31</sub>	Details of the methodology for Trip Actuating Device Operational Testing of the Reactor Trip Breakers, Reactor Trip Bypass Breakers, and the associated shunt trip and undervoltage trip mechanisms.	Bases	Bases Control Program
ITS SECTION: 3.4 REACTOR COOLANT SYSTEM (RCS)			
R <sub>1</sub>	The requirement for a minimum of one pressurizer Code safety valve to be Operable in Modes 4 and 5.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>2</sub>	The requirement to ensure that the Reactor Coolant System chemistry is maintained within prescribed limits to minimize corrosion.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>3</sub>	The requirement to ensure pressurizer temperature maximum heatup and cooldown rate limits to prevent nonductile failure and to assure compatibility of operation with the fatigue analysis.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>4</sub>	The requirement to ensure the structural integrity of ASME Code Class 1, 2, and 3 components are maintained in accordance with Section XI of the ASME Code.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>5</sub>	The requirement to ensure the Operability and closure of at least one reactor head vent path whenever the unit is in Mode 1, 2, 3, or 4.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>1</sub>	Descriptive information associated with the operating verification of the reactor coolant loops.	Bases	Bases Control Program
LA <sub>2</sub>	Details associated with the components that constitutes Reactor Coolant System loop Operability.	Bases	Bases Control Program
LA <sub>3</sub>	Descriptive information associated with the operating verification of the Reactor Coolant System and Residual Heat Removal loops.	Bases	Bases Control Program

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 8 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.4 REACTOR COOLANT SYSTEM (RCS) (continued)</b>			
LA <sub>4</sub>	Details associated with the components that constitutes Reactor Coolant System and Residual Heat Removal loop Operability.	Bases	Bases Control Program
LA <sub>6</sub>	Details associated with the testing method used to demonstrate pressurizer heater Operability.	Bases	Bases Control Program
LA <sub>7</sub>	Details associated with the testing method for verifying reactor coolant gross specific activity.	Bases	Bases Control Program
LA <sub>8</sub>	The requirement for Channel Calibration testing of the Power Operated Relief Valve actuation instrumentation.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>9</sub>	Details associated with for performance of the action to analyze containment atmosphere (analyzed for gaseous and particulate radioactivity).	Bases	Bases Control Program
LA <sub>10</sub>	Details associated with the Reactor Coolant System Pressure Isolation Valve test performance for various unit conditions.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>11</sub>	The listing of Reactor Coolant System Pressure Isolation Valves.	Bases	Bases Control Program
LA <sub>14</sub>	Details associated with the specific demonstration ensuring Safety Injection and charging pumps are incapable of injecting into the Reactor Coolant System during Low Temperature Overpressure Protection conditions.	Bases	Bases Control Program
LA <sub>15</sub>	Details associated with specific valve identifiers for the Residual Heat Removal suction valves and Residual Heat Removal suction relief valves.	Bases	Bases Control Program
LA <sub>17</sub>	Details associated with descriptive information for operation of four reactor coolant loops.	Bases	Bases Control Program
LA <sub>18</sub>	Details associated with the method for determining the Reactor Coolant System total flow rate.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>19</sub>	The requirement to perform a Channel Calibration every 18 months on the Reactor Coolant System total flow rate indicators.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>20</sub>	Details associated with a pressurizer temperature notation associated with the Reactor Coolant System Pressure and Temperature LCO limits.	Bases	Bases Control Program

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 9 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
ITS SECTION: 3.4 REACTOR COOLANT SYSTEM (RCS) (continued)			
LA <sub>21</sub>	Descriptive information associated with the condition of the operating steam generators during the determination of Reactor Coolant System operational leakage.	Bases	Bases Control Program
LA <sub>22</sub>	Details associated with the performance of an engineering evaluation for determining the acceptability of the Reactor Coolant System for continued operation.	Bases	Bases Control Program
LA <sub>23</sub>	The requirement to verify that the oil separator portion of the containment floor drain collection sump has been filled to the level of the overflow to the containment floor drain collection weir box.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>24</sub>	The requirements for determining Reactor Coolant System leakages through monitoring of the containment atmosphere radioactivity, reactor cavity sump discharge, the containment floor drain sump discharge and inventory, and the reactor head flange leakoff.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>25</sub>	The requirement to submit a special report in the event that either the Power Operated Relief Valves, the Residual Heat Removal suction relief valves, or the Reactor Coolant System vents are used to mitigate a Reactor Coolant System pressure transient.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>26</sub>	The requirement to verify Reactor Coolant System Pressure Isolation Valve leakages for affected valves following any maintenance, repair, or replacement work prior to returning the valve to service.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>27</sub>	Details associated with specific equipment identifiers for the containment atmosphere gaseous and particulate radioactivity monitors.	Bases	Bases Control Program
LA <sub>28</sub>	Details associated with the method used to determine Containment Floor Drain and Reactor Cavity Sump Flow Monitoring System Operability.	Bases	Bases Control Program
LA <sub>29</sub>	The requirement to verify the Reactor Coolant System average temperature limits once per 30 minutes when the reactor is critical and the Reactor Coolant System $T_{avg}$ is $< 557^{\circ}\text{F}$ with the $T_{avg}-T_{ref}$ Deviation Alarm not reset.	TRM <sup>(1)</sup>	10CFR50.59

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 10 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS)</b>			
R <sub>1</sub>	The requirement to ensure, when in Mode 5 and 6 with the pressurizer level less than or equal to 5 percent, that a diverse means of introducing makeup water from the Refueling Water Storage Tank to the Reactor Coolant System are available in the event that a loss of inventory or loss of forced circulation occurs which results in a loss of decay heat removal.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>1</sub>	Details associated with the method of removing power from each Emergency Core Cooling System accumulator valve operator.	Bases	Bases Control Program
LA <sub>2</sub>	The requirement to perform a Channel Calibration every 18 months for the accumulator water level and pressure channel.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>3</sub>	Details associated with describing an Operable Emergency Core Cooling System subsystem and flow path.	Bases	Bases Control Program
LA <sub>4</sub>	The requirements (1) to prepare and submit a Special Report to the NRC within 90 days whenever the Emergency Core Cooling System is actuated and injects water into the Reactor Coolant System and (2) to provide the usage factor for each affected Safety Injection nozzle whenever its value exceeds 0.70.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>5</sub>	Details associated with specific equipment identifiers and descriptive information associated with isolation of Emergency Core Cooling System flow paths and closure of isolation valves during the performance of pressure isolation valve surveillance testing.	Bases	Bases Control Program
LA <sub>6</sub>	The requirement to ensure accumulator Operability during Mode 3 with pressurizer pressure < 1000 psig whenever isolation valves SI8809A or SI8809B are closed.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>7</sub>	Details describing the method used to ensure that the Emergency Core Cooling System piping is full of water.	Bases	Bases Control Program
LA <sub>8</sub>	The requirement to ensure, by a visual inspection, that no loose debris is present in the containment.	TRM <sup>(1)</sup>	10CFR50.59

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 11 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.5 EMERGENCY CORE COOLING SYSTEMS (ECCS) (continued)</b>			
LA <sub>9</sub>	Details associated with the method for performing the Emergency Core Cooling System automatic valve actuation and automatic pump start surveillance tests.	Bases	Bases Control Program
LA <sub>10</sub>	The listing of Emergency Core Cooling System pumps.	Bases	Bases Control Program
LA <sub>11</sub>	Details describing the pump acceptance criteria and test methods associated with the Emergency Core Cooling System pump performance surveillance test.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>12</sub>	The requirement to ensure the correct position of each mechanical stop for certain Emergency Core Cooling System throttle valves within 4 hours of completing a valve stroke operation or valve maintenance.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>13</sub>	The requirement to perform a flow balance test to the Emergency Core Cooling System subsystems following the completion of modifications that alter the subsystem flow characteristics.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>14</sub>	Details associated with position stops of the Emergency Core Cooling System throttle valves.	Bases	Bases Control Program
LA <sub>15</sub>	Details describing the Operability requirements associated with the Refueling Water Storage Tank vent path.	Bases	Bases Control Program
<b>ITS SECTION: 3.6 CONTAINMENT SYSTEMS</b>			
LA <sub>1</sub>	The containment isolation valve component listing has been relocated consistent with the direction of Generic Letter 91-08, "Removal Of Component Lists From Technical Specifications."	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>2</sub>	Details associated with the Operability of the containment air lock interlock function.	Bases	Bases Control Program
LA <sub>3</sub>	Details associated with the method of determining average containment air temperature.	Bases	Bases Control Program
LA <sub>4</sub>	Details associated with the method to ensure that the 48-inch purge valves are "sealed closed" and assured of not inadvertently opening.	Bases	Bases Control Program



**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 12 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.6 CONTAINMENT SYSTEMS (continued)</b>			
LA <sub>6</sub>	Details associated with design attributes of the Containment Spray System and details describing the minimum requirements for an Operable flow path.	Bases	Bases Control Program
LA <sub>7</sub>	Details associated with the required developed head for the containment spray pump and test flow path during performance of the Surveillance.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>8</sub>	Details associated with the verification that the spray nozzles are unobstructed and the method used in performing this verification.	Bases	Bases Control Program
LA <sub>9</sub>	Details associated with describing the minimum requirements of an Operable Spray Additive System flow path.	Bases	Bases Control Program
LA <sub>10</sub>	Details associated with the acceptance criteria for the flow rate verification surveillance of the Spray Additive System.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>11</sub>	Details associated with the design attributes of the containment cooling fans and details describing the minimum requirements for an Operable cooling train.	Bases	Bases Control Program
LA <sub>12</sub>	Details associated with the method of conducting the containment cooling fan system surveillance.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>13</sub>	Details associated with the design attributes of the Hydrogen Recombiner System.	Bases	Bases Control Program
LA <sub>14</sub>	Details of 1) rate of heater sheath temperature increase and power consumption; 2) channel calibration of instrumentation and controls; 3) attributes of a visual examination; and 4) required heater phase resistance associated with specific acceptance criteria for hydrogen recombiner testing.	Bases/TRM <sup>(1)</sup>	Bases Control Program/ 10CFR50.59
LA <sub>15</sub>	The requirement to demonstrate Operability after maintenance or repair on an isolation valve.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>16</sub>	Details of the acceptance criteria for leakage rate tests associated with the containment purge valves.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>17</sub>	Details associated with the method for isolating the 48-inch purge valves.	Bases	Bases Control Program



**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**

(page 13 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.7 PLANT SYSTEMS</b>			
R <sub>1</sub>	The requirement to ensure that the temperatures of the reactor and secondary coolants in the steam generators are > 70°F when the pressure of either coolant in the steam generator is > 200 psig.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>3</sub>	The requirement to ensure that leakage from byproduct, source and special nuclear material sources will not exceed allowable intake values.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>4</sub>	The requirement to ensure that the temperature in areas of the plant which contain safety related equipment does not exceed its environmental qualification temperature during normal operation.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>1</sub>	Details relating to the lift setting pressure of the Main Steam Safety Valves.	Bases	Bases Control Program
LA <sub>2</sub>	Details associated with design attributes of an Operable Auxiliary Feedwater System and details describing the minimum requirements for an Operable Auxiliary Feedwater System.	Bases	Bases Control Program
LA <sub>3</sub>	Details associated with the required developed head for the Auxiliary Feedwater pumps during Surveillances.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>4</sub>	The requirements associated with the Operability of the diesel portion of the diesel driven Auxiliary Feedwater pump.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>5</sub>	Details associated with the verification of Operability of the Essential Service Water System when used as a backup to the Condensate Storage Tank for the Auxiliary Feedwater System.	Bases	Bases Control Program
LA <sub>6</sub>	The requirements associated with the Operability of the Component Cooling Water System.	Bases	Bases Control Program
LA <sub>7</sub>	The requirement associated with a monthly surveillance demonstrating the Operability of the Component Cooling Water System pumps.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>8</sub>	Details associated with the common Component Cooling Water System pump tests have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>9</sub>	The requirement associated with the 31-day verification of Operability of the Component Cooling Water System heat exchanger inlet and outlet valves.	TRM <sup>(1)</sup>	10CFR50.59

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 14 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.7 PLANT SYSTEMS (continued)</b>			
LA <sub>11</sub>	Details associated with design attributes of the Essential Service Water System have been relocated.	Bases	Bases Control Program
LA <sub>12</sub>	Details associated with the Operability of the opposite-unit Essential Service Water System pump and crosstie and details associated with specific equipment identifiers for the crosstie valves have been relocated.	Bases	Bases Control Program
LA <sub>13</sub>	The requirements associated with demonstrating the availability of the opposite-unit Essential Service Water System to support unit-specific operation have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>14</sub>	For Braidwood Station only, the requirement associated with the performance of a hydrographic survey every 18 months to verify no degradation on the Ultimate Heat Sink.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>15</sub>	For Braidwood Station only, details associated with the method for determining Ultimate Heat Sink temperature have been relocated.	Bases	Bases Control Program
LA <sub>16</sub>	Details associated with design attributes of the Nonaccessible Area Exhaust Filter Plenum Ventilation System and the method for demonstrating Operability of the Nonaccessible Area Exhaust Filter Plenum Ventilation System have been relocated.	Bases/TRM <sup>(1)</sup>	Bases Control Program/ 10CFR50.59
LA <sub>17</sub>	Details associated with design attributes and the methods of performing surveillances of the Control Room Ventilation Filtration System and Control Room Ventilation Temperature Control System have been relocated.	Bases	Bases Control Program
LA <sub>18</sub>	Details associated with potential causes of Control Room Ventilation Temperature Control System inoperability have been relocated.	Bases	Bases Control Program
LA <sub>19</sub>	Details of the recirculation flow rate acceptance criteria associated with the Control Room Ventilation Filtration System positive nominal pressure surveillances have been relocated.	TRM <sup>(1)</sup>	10CFR50.59

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 15 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.7 PLANT SYSTEMS (continued)</b>			
LA <sub>21</sub>	The requirements associated with the Fuel Handling Building Exhaust Filter Plenum Ventilation System Operability during the movement of new fuel or the movement of heavy loads over irradiated fuel in the fuel handling building have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>22</sub>	Details associated with the alignment of other systems during the performance of the Fuel Handling Building Exhaust Filter Plenum Ventilation System surveillance have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>23</sub>	Details associated with design attributes, the methods of performing surveillances, and the method for demonstrating Operability of the of the Fuel Handling Building Exhaust Filter Plenum Ventilation System have been relocated.	Bases/TRM <sup>(1)</sup>	Bases Control Program/ 10CFR50.59
LA <sub>24</sub>	For Byron Station only, details associated with the method of verifying the Operability of the deep well pumps and the requirements associated with deep well pumps as related to the Ultimate Heat Sink Operability have been relocated.	Bases/TRM <sup>(1)</sup>	Bases Control Program/ 10CFR50.59
LA <sub>25</sub>	For Byron Station only, the requirements associated with the Operability of the diesel portions of the Essential Service Water System diesel driven makeup pumps have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>26</sub>	For Byron Station only, the requirements associated with the Operability of the Ultimate Heat Sink basin level switches providing the automatic start signal for the makeup pumps on low basin level have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>27</sub>	For Byron Station only, the requirement associated with visually inspecting the cooling tower fill to verify no degradation of the Ultimate Heat Sink.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>28</sub>	For Byron Station only, details associated with design attributes of performing the Ultimate Heat Sink fan surveillance have been relocated.	Bases/TRM <sup>(1)</sup>	Bases Control Program/ 10CFR50.59
LA <sub>29</sub>	For Byron Station only, details associated with Ultimate Heat Sink Operability have been relocated.	Bases	Bases Control Program
LA <sub>30</sub>	For Byron Station only, details associated with the Ultimate Heat Sink basin level switches have been relocated.	Bases	Bases Control Program

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 16 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 3.7 PLANT SYSTEMS (continued)</b>			
LA <sub>31</sub>	For Byron Station only, the requirements associated with the National Weather Service forecasts of river level and tornadoes to ensure Ultimate Heat Sink Operability have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>32</sub>	For Byron Station only, the requirement to provide a special report to the NRC if any basin level switch is inoperable for more than 30 days.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>33</sub>	For Byron Station only, details associated with river level and flow rate to ensure Ultimate Heat Sink Operability have been relocated.	Bases	Bases Control Program
LA <sub>34</sub>	Details associated with the listing of Feedwater Isolation Valves have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>35</sub>	The requirement associated with demonstrating Operability of a Feedwater Isolation Valve after maintenance or repair.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>37</sub>	Details associated with the method of isolating the Feedwater Isolation Valves have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>38</sub>	Details associated with placing an Operable Nonaccessible Area Exhaust Filter Plenum Ventilation System train in standby for the purpose of testing an inoperable train have been relocated.	Bases	Bases Control Program
LA <sub>39</sub>	The requirements associated with the Operability and testing of snubbers have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
<b>ITS SECTION: 3.8 ELECTRICAL POWER SYSTEMS</b>			
R <sub>1</sub>	The requirements associated with the Operability and testing of containment penetration conductor overcurrent protective devices to minimize the potential for a fault in a component inside containment, or in cabling which penetrates the containment have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>2</sub>	The requirements associated with the Operability and testing of motor-operated valves thermal overload protection devices which minimize the potential for an improper setting of a thermal overload have been relocated.	TRM <sup>(1)</sup>	10CFR50.59

## CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS

(page 17 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
ITS SECTION: 3.8 ELECTRICAL POWER SYSTEMS (continued)			
LA <sub>1</sub>	Details associated with defining the two incoming offsite AC power sources, design features of the diesel generators, and listing of the diesel generator fuel oil volume requirements have been relocated.	Bases	Bases Control Program
LA <sub>2</sub>	Details associated with start signal options that can be used for the monthly diesel generator start surveillance have been relocated.	Bases	Bases Control Program
LA <sub>3</sub>	Details associated with the explicit verification that the diesel generator is aligned to provide standby power have been relocated.	Bases	Bases Control Program
LA <sub>4</sub>	Details associated with the allowance to perform a pre-lube prior to the performance of a diesel generator surveillance have been relocated.	Bases	Bases Control Program
LA <sub>5</sub>	Details associated with the specific single largest post-accident load for the diesel generator single load rejection surveillance have been relocated.	Bases	Bases Control Program
LA <sub>6</sub>	Details associated with the acceptable load range for the diesel generator load run surveillance have been relocated.	Bases	Bases Control Program
LA <sub>7</sub>	The requirements associated with a 10-second start and confirmation of operating voltage and frequency during the 24-hour diesel generator surveillance have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>8</sub>	Details associated with the design for auto-connected loads to each diesel generator have been relocated.	UFSAR	10CFR50.59
LA <sub>9</sub>	Details associated with the acceptance criteria for correcting specific gravity for electrolyte temperature and level to ensure battery operability have been relocated.	Bases	Bases Control Program
LA <sub>10</sub>	The requirement associated with verifying the ability to cross-tie the "A" diesel generator to the respective opposite unit's 4 kV ESF bus to ensure the ability to power a unit-specific auxiliary feedwater pump from the opposite-unit diesel generator.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>11</sub>	The requirement associated with verifying the diesel generator lockout features to prevent diesel generator starting only when required.	TRM <sup>(1)</sup>	10CFR50.59



## CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS

(page 18 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
ITS SECTION: 3.8 ELECTRICAL POWER SYSTEMS (continued)			
LA <sub>12</sub>	Details associated with the method for cleaning the diesel fuel storage tank and the requirement to perform a pressure test on the diesel generator fuel oil system piping have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>13</sub>	The requirements associated with crane operation with loads over the spent fuel pool following a loss of power sources have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>14</sub>	Details associated with design features and Operability of the DC electric sources, Instrument Buses and the Instrument Bus inverters have been relocated.	Bases/UFSAR	Bases Control Program/ 10CFR50.59
LA <sub>15</sub>	Details associated with battery degradation criteria have been relocated.	Bases	Bases Control Program
LA <sub>16</sub>	The requirement associated with demonstrating Operability after modifications which could affect diesel generator interdependence.	TRM <sup>(1)</sup>	10CFR50.59
ITS SECTION: 3.9 REFUELING OPERATIONS			
R <sub>2</sub>	The requirement associated with the communication between the control room and the refueling station to ensure that any abnormal change in the facility status observed on the control room instrumentation can be communicated to the refueling station personnel.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>3</sub>	The requirements to (1) ensure that the refueling machine and auxiliary hoist will have sufficient load capacity for their intended purpose and will be used correctly during refueling and (2) ensure that the core internals and reactor vessel are protected from excessive lifting force during refueling operations have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
R <sub>4</sub>	The requirements to ensure that loads in excess of the nominal weight of one fuel assembly containing a Rod Control Cluster Assembly, plus the weight of the fuel handling tool, will not be moved over other fuel assemblies stored in the spent fuel storage racks have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>1</sub>	The specific value associated with the minimum boron concentration.	COLR <sup>(1)</sup>	10CFR50.59



## CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS

(page 19 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
ITS SECTION: 3.9 REFUELING OPERATIONS (continued)			
LA <sub>2</sub>	The requirement associated with maintaining boron concentration at a limit to ensure a core $k_{eff}$ of $\leq 0.95$ .	Bases	Bases Control Program
LA <sub>3</sub>	Details associated with the method of restoring boration flow rate and minimum boron concentration have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>4</sub>	The specific listing of unborated water source isolation valves.	Bases	Bases Control Program
LA <sub>5</sub>	Details associated with the methods for verifying that the unborated water source isolation valves are closed and secured in position have been relocated.	Bases	Bases Control Program
LA <sub>6</sub>	The requirement associated with a time limit on reactor subcriticality prior to the movement of irradiated fuel assemblies.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>7</sub>	Details associated with the regions required to maintain minimum boron concentration have been relocated.	Bases	Bases Control Program
LA <sub>8</sub>	Details associated with the analysis used for determining boron concentration have been relocated.	Bases	Bases Control Program
LA <sub>9</sub>	Details associated with the Operability requirements of the source range neutron flux monitors and the requirement that each monitor provide visual indication have been relocated.	Bases	Bases Control Program
LA <sub>10</sub>	The requirements associated with Residual Heat Removal loop flow rate and Reactor Coolant System temperature for the performance of Residual Heat Removal loop testing have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>11</sub>	The requirements associated with reactor vessel water level during movement of nonirradiated fuel assemblies within the containment have been relocated.	TRM <sup>(1)</sup>	10CFR50.59

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 20 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 4.0 DESIGN FEATURES</b>			
LA <sub>1</sub>	Details associated with the containment design, Reactor Coolant System design, and the meteorological tower location have been relocated.	UFSAR	10CFR50.59
LA <sub>2</sub>	Details associated with the fuel assembly design and control rod assembly design have been relocated.	UFSAR	10CFR50.59
LA <sub>3</sub>	Details associated with the Exclusion Area Boundary, Low Population Zone, and unrestricted areas and site boundary for radioactive gaseous and liquid effluents have been relocated.	UFSAR	10CFR50.59
<b>ITS SECTION: 5.0 ADMINISTRATIVE CONTROLS</b>			
LA <sub>1</sub>	Details associated with the minimum requirements for shift crew composition have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>2</sub>	Details associated with functional requirements of the Independent Safety Engineering Group have been relocated.	UFSAR Chapter 17	10CFR50.54(a)
LA <sub>3</sub>	The requirements associated with facility staff training and retraining have been relocated.	UFSAR	10CFR50.59
LA <sub>4</sub>	The requirements associated with plant review and submittal of a reportable event have been relocated.	UFSAR Chapter 17	10CFR50.54(a)
LA <sub>5</sub>	The requirement associated with establishing procedures to implement the Process Control Program.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>6</sub>	The requirements associated with implementation of the technical review and control program for procedure changes have been relocated.	UFSAR Chapter 17	10CFR50.54(a)
LA <sub>7</sub>	The requirements associated with implementation of the in-plant radiation monitoring program have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>8</sub>	The requirements associated with implementation of the Offsite Dose Calculation Manual and the radiological environmental monitoring program have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>9</sub>	The requirement associated with submittal of a report to the NRC following specific activity limits being exceeded.	TRM <sup>(1)</sup>	10CFR50.59

**CTS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 21 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
<b>ITS SECTION: 5.0 ADMINISTRATIVE CONTROLS (continued)</b>			
LA <sub>10</sub>	The requirements associated with the retention of records related to activities affecting quality have been relocated.	UFSAR Chapter 17	10CFR50.54(a)
LA <sub>11</sub>	The requirement associated with establishing procedures for personnel radiation protection consistent with 10 CFR 20.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>12</sub>	Details associated with test and frequency requirements for the Control Room Ventilation, the Nonaccessible Area Exhaust Filter Plenum Ventilation, and the Fuel Handling Building Exhaust Filter Plenum Ventilation Systems have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>13</sub>	Details associated with functional requirements of the Onsite Review and Investigative Function have been relocated.	UFSAR Chapter 17	10CFR50.54(a)
LA <sub>14</sub>	The requirements associated with containment vessel structural integrity have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>15</sub>	The requirements associated with diesel fuel oil testing have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>16</sub>	The requirements associated with component cyclic or transient limits have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>17</sub>	For Byron Station only, the requirements associated with ASME Section XI testing requirements have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>18</sub>	The requirement associated with submittal of a startup report to the NRC.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>19</sub>	Details associated with the attributes of containment Operability and the requirements associated with Appendix J testing of sealing mechanisms for penetrations have been relocated.	ITS 5.5.16 Program <sup>(1)</sup>	10CFR50.59
LA <sub>20</sub>	The requirements associated with explosive gas and storage tank radioactivity monitoring have been relocated.	ITS 5.5.12 Program <sup>(1)</sup>	10CFR50.59
LA <sub>22</sub>	Details associated with the pressure and temperature limits, heatup and cooldown rates, reactor vessel material surveillance schedule, and Power Operated Relief Valve lift setpoints have been relocated.	PTLR <sup>(1)</sup>	10CFR50.59
LA <sub>23</sub>	The requirements associated with reactor coolant pump flywheel inspections have been relocated.	ITS 5.5.7 Program <sup>(1)</sup>	10CFR50.59

**ITS REQUIREMENT RELOCATED TO OTHER LICENSEE CONTROLLED DOCUMENTS**  
(page 22 of 22)

DOC	ISSUE RELOCATED	DOCUMENT	CONTROL
ITS SECTION: 5.0 ADMINISTRATIVE CONTROLS (continued)			
LA <sub>24</sub>	The requirements associated with containment leakage and air lock testing have been relocated.	ITS 5.5.16 Program <sup>(1)</sup>	10CFR50.59
LA <sub>25</sub>	The requirements associated with battery inspections have been relocated.	ITS 5.5.17 Program <sup>(1)</sup>	10CFR50.59
LA <sub>26</sub>	The requirements associated with the responsibility of the Shift Engineer for directing the control room command function and the daily operations of the facility have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>27</sub>	Details associated with establishing and implementing procedures to limit the working hours of facility staff who perform safety-related functions have been relocated.	TRM <sup>(1)</sup>	10CFR50.59
LA <sub>28</sub>	Details associated with plant specific management position titles have been relocated.	UFSAR Chapter 17	10CFR50.54(a)

Table Notation:

(1) To be incorporated by reference into the UFSAR.

ATTACHMENT 8

ENVIRONMENTAL ASSESSMENT  
FOR PROPOSED CHANGES  
TO APPENDIX A, TECHNICAL SPECIFICATIONS,  
OF FACILITY OPERATING LICENSES  
NPF-37, NPF-66, NPF-72, AND NPF-77

**ENVIRONMENTAL ASSESSMENT  
FOR PROPOSED CHANGES TO  
APPENDIX A, TECHNICAL SPECIFICATIONS,  
OF FACILITY OPERATING LICENSES  
NPF-37, NPF-66, NPF-72, AND NPF-77**

(page 1 of 1)

Commonwealth Edison Company (ComEd) has evaluated this proposed license amendment request against the criteria for identification of licensing and regulatory actions requiring environmental assessment in accordance with Title 10, Code of Federal Regulations, Part 51, Section 21 (10 CFR 51.21). ComEd has determined that this proposed license amendment request meets the criteria for a categorical exclusion set forth in 10 CFR 51.22(c)(9). This determination is based upon the following:

1. The proposed licensing action involves the issuance of an amendment to a license for a reactor pursuant to 10 CFR 50 which changes a requirement with respect to installation or use of a facility component located within the restricted area, as defined in 10 CFR 20, or which changes an inspection or a surveillance requirement;
2. This proposed license amendment request involves no significant hazards considerations as demonstrated in the enclosure to this letter;
3. There is no significant change in the types or significant increase in the amounts of any effluent that may be released offsite; and
4. There is no significant increase in individual or cumulative occupational radiation exposure.

Therefore, pursuant to 10 CFR 51.22(b), neither an environmental impact statement nor an environmental assessment is necessary for this proposed license amendment request.