NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Browns Ferry - Unit 2	0 5 0 0 0 2 6 0	1 OF 0 2

TITLE (4)
Containment Isolation Due to Breaker Failure

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 6	2 7	8 5	8 5	0 0 7	0 0	0 7	2 6	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 0 0		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
R. C. Steele	2 0 5 7 2 9 - 3 5 8 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
A V	E D	B K R	G 0 8 0	Yes							
B B	E D	B K R	G 0 8 0	Yes							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While attempting to transfer a unit two 480 V shutdown board to its alternate power source, the alternate feeder breaker failed to close resulting in loss of power to the board. Loss of the shutdown board resulted in a secondary containment isolation and trip of one reactor protection system (RPS) motor generator (MG) set.

The operator reclosed the normal breaker which restored power to the shutdown board and reset the isolation. The breaker failed due to loosened bolts holding the gearbox assembly. The problem was corrected, and the breaker returned to service. An investigation is being conducted on the response of the RPS MG set control circuit.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Units 1 and 2 were in a refueling outage, and unit 3 was in an extended maintenance outage.

On June 27, 1985, operators attempted to transfer 480 V shutdown board 2A (ED) to its alternate feeder for maintenance on its normal feeder transformer (TS2A). The alternate feeder breaker (BKR) failed to close, resulting in a secondary containment isolation (JM) and loss of reactor protection system (RPS) motor generator (MG) set "B" (JC).

The loss of power to components fed off the shutdown board resulted in a reactor zone and refuel zone isolation. Standby gas treatment and control room emergency ventilation also initiated. The operator reclosed the normal breaker, restoring power to the shutdown board, and the isolation signal was reset. During the event, the affected containment isolation system components functioned as designed with no adverse effects noted.

Investigation into the breaker failure revealed that the bolts holding the charging gearbox assembly had loosened. The bolts were tightened, and a complete maintenance service was performed on the breaker. During subsequent functional testing of the alternate breaker, the normal supply breaker would not open, and the tripping mechanism appeared to be binding. Maintenance was also performed on the normal feeder breaker before returning it to service.

At the time of the trip, both RPS MG sets were being fed off 480 V shutdown board 2A. An investigation is continuing to determine why RPS MG set "B" tripped in this event.

Responsible Plant Section - N/A

Previous Events - None

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

July 26, 1985

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 2 - DOCKET NO. 50-260 - FACILITY OPERATING LICENSE DPR-52 - REPORTABLE OCCURRENCE REPORT BFRO-50-260/85007

The enclosed report provides details concerning containment isolation due to breaker failure. This report is submitted in accordance with 10 CFR 50.73(a)(2)(iv).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

for R. L. Lewis
Acting Plant Manager
Browns Ferry Nuclear Plant

Enclosures
cc (Enclosures):
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U. S. Nuclear Regulatory Commission
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NRC Resident Inspector, BFN

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