

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Fermi-2DOCKET NUMBER (2)
0 5 0 0 0 3 4 1 1 OF 0 2TITLE (6)
HPCI Isolation During Surveillance Testing

EVENT DATE (8)			LER NUMBER (9)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (5)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER (5)
08	27	85	85	054		00	09	24	85		0 5 0 0 0

OPERATING MODE (3)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 6 (Check one or more of the following) (11)																														
2	<table border="1"><tr><td>20.402(a)</td><td>20.408(a)</td><td>X</td><td>60.73(a)(2)(iv)</td><td>78.71(b)</td></tr><tr><td>20.408(a)(1)(i)</td><td>60.36(a)(1)</td><td></td><td>60.73(a)(2)(v)</td><td>78.71(a)</td></tr><tr><td>20.408(a)(1)(ii)</td><td>60.36(a)(2)</td><td></td><td>60.73(a)(2)(vi)</td><td>OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td></tr><tr><td>20.408(a)(1)(iii)</td><td>60.73(a)(2)(i)</td><td></td><td>60.73(a)(2)(vii)(A)</td><td></td></tr><tr><td>20.408(a)(1)(iv)</td><td>60.73(a)(2)(ii)</td><td></td><td>60.73(a)(2)(vii)(B)</td><td></td></tr><tr><td>20.408(a)(1)(v)</td><td>60.73(a)(2)(iii)</td><td></td><td>60.73(a)(2)(viii)</td><td></td></tr></table>	20.402(a)	20.408(a)	X	60.73(a)(2)(iv)	78.71(b)	20.408(a)(1)(i)	60.36(a)(1)		60.73(a)(2)(v)	78.71(a)	20.408(a)(1)(ii)	60.36(a)(2)		60.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.408(a)(1)(iii)	60.73(a)(2)(i)		60.73(a)(2)(vii)(A)		20.408(a)(1)(iv)	60.73(a)(2)(ii)		60.73(a)(2)(vii)(B)		20.408(a)(1)(v)	60.73(a)(2)(iii)		60.73(a)(2)(viii)	
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20.408(a)(1)(v)	60.73(a)(2)(iii)		60.73(a)(2)(viii)																												

LICENSEE CONTACT FOR THIS LER (12)
NAME
L. P. Bregni, Compliance EngineerTELEPHONE NUMBER
3 1 3 5 8 6 - 5 3 1 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) X NO

EXPECTED SUBMISSION DATE (15)

MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On August 27, 1985 with the plant in Operational Condition 2 and reactor power at 4.6 percent, an isolation of the High Pressure Coolant Injection (HPCI) system occurred at 0055 hours during surveillance testing of HPCI & RCIC Room Area Temperature Channel B. The surveillance procedure requires step being performed lifting the thermocouple input leads from Reactor Core Isolation Cooling (RCIC) switch module E51-N602B. Test personnel incorrectly selected module E41-N602B (HPCI) instead. When they lifted the thermocouple leads on module E41-N602B, the HPCI outboard valve isolation logic tripped. The Control Room Nuclear Supervising Operator determined that the isolation was caused by the testing personnel error, then reset the HPCI isolation logic and returned HPCI to Standby. Test personnel relanded the lifted leads. To prevent recurrence of this error, diagrams showing the location and identification of the switch modules have been added to the affected surveillance procedures (44.020.156, .157, .158, .159, .227, .228, .229, .230). This event has been discussed with all Instrument & Controls test personnel.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Fermi-2	DOCKET NUMBER (2) 05000341	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		85	054	00	02	OF	02

TEXT (If more space is required, use additional NRC Form 366A (17))

On August 27, 1985, with the plant in Operational Condition 2 and reactor power at 4.6 percent, an isolation of the High Pressure Coolant Injection System (HPCI) occurred at 0055 hours during surveillance test 44.020.228, "NSSSS- HPCI & RCIC Room Area Temperature Channel B Functional Test". Test personnel were performing surveillances on area temperature switch modules inside Relay Room Panel H11-P614.

The surveillance procedure requires lifting the thermocouple input leads from module E51-N602B (Reactor Core Isolation Cooling System). From the front of Panel H11-P614, test personnel correctly identified the module to be tested. When they entered the rear of the panel to lift the leads on that module, they incorrectly selected module E41-N602B (HPCI) instead. When they lifted the thermocouple leads on module E41-N602B, the HPCI primary containment outboard isolation valve logic tripped as designed.

Outboard isolation valve E41-F600 in the 1-inch HPCI steam supply bypass line closed as designed. The HPCI steam supply outboard isolation valve (E41-F003) and the HPCI pump suction outboard isolation valve (E41-F041) received a close signal but were already closed. (These valves are normally closed in the HPCI standby lineup). Actuation of the HPCI primary containment isolation logic is a reportable ESF actuation.

The Control Room Nuclear Supervising Operator (NSO) determined that the isolation was caused by the testing personnel error. Test personnel relanded the lifted leads, and the NSO reset the HPCI isolation logic and returned HPCI to Standby.

The cause of this event was personnel error in selecting the wrong module (E41-N602B) inside panel H11-P614. To prevent recurrence of this error, diagrams showing the location and identification of the switch modules from the rear of the panel have been added to the affected surveillance procedures (44.020.156, .157, .158, .159, .227, .228, .229, .230). This event has been discussed with all Instrument & Controls test personnel.



Robert S. Lenart
Plant Manager

Fermi-2
6400 North Dixie Highway
Newport, Michigan 48166
(313) 586-5201

September 24, 1985
NP850100



Nuclear
Operations

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Reference: Fermi 2
NRC Docket No. 50-341
NRC Operating License No. NPF-43

Subject: Transmittal of Licensee
Event Report 85-054

Please find enclosed LER No. 85-054-00, dated September 24, 1985, for a reportable event which occurred on August 27, 1985. As indicated below, a copy of this LER is being sent to the Administrator Region III.

If you have any questions, please contact us.

Sincerely,

R. S. Lenart
Plant Manager

Enclosure: NRC Forms 366, 366A

cc: P.M. Byron
M.D. Lynch

Regional Administrator
USNRC Region III
799 Roosevelt Rd.
Glen Ellyn, IL 60137

Director/Coordinator
Monroe City-County Office of Civil Preparedness
965 South Raisinville Road
Monroe, MI 48161

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