

MONTHLY OPERATING REPORT - JULY 1985

At the beginning of the report period, Oyster Creek was operating at approximately 645 MWe, with load restricted by intake temperature and its effect on condenser vacuum.

On July 1, Core Spray System No. 2 snubber hanger No. 2138 was found to be cracked. Repairs were completed July 3, satisfying a 72-hour Technical Specification limit. Core Spray System No. 2 was tested and returned to service July 4.

On July 8, an automatic reactor scram occurred due to low condenser vacuum. The low vacuum condition was caused by a cracked casing on No. 1-1 steam jet air ejector (SJAE) drain tank pump which resulted in excessive drain tank levels and flooding of the SJAEs. At the time of the scram, No. 1-2 SJAE drain tank pump was tagged out-of-service in preparation for seal replacement.

Following completion of repairs and surveillances, reactor startup commenced on July 9. No. 1-1 SJAE drain tank pump remained out of service pending replacement. Reactor and generator startup was completed and power subsequently increased to approximately 515 MWe by July 10.

On July 13, power was reduced to 350 MWe to facilitate cleaning of No. 1-2 Turbine Building Closed Cooling Water (TBCCW) heat exchanger. Power was increased to 500 MWe later that day subsequent to cleaning the heat exchanger.

On July 15, after removing Containment Spray System II from service to determine the cause of increased emergency service water (ESW) differential pressure, ESW pump 52A (System I) bearing temperature increased during operation. The pump was considered inoperable and in accordance with Tech Spec requirements, a reactor shutdown commenced. Shutdown was terminated after returning System II to service and verifying system operability.

Power was maintained at approximately 500 MWe with load restricted by high drain tank levels due to the degraded performance of No. 1-2 SJAE drain tank pump. Subsequent to replacement of No. 1-1 SJAE drain tank pump, power was increased to 615 MWe on July 16.

On July 17, inspections of Containment Spray System II heat exchangers revealed pieces of black brittle material suspected to be interior coating from ESW system piping. Upon completion of inspections and cleaning, the heat exchangers were returned to service to conduct a test run.

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MONTHLY OPERATING REPORT - JULY 1985

On July 20, Containment Spray System II was removed from service to facilitate a heat exchanger inspection subsequent to a test run wherein a high heat exchanger differential pressure condition developed. ESW pump 52A (System I) failed its associated surveillance test due to low discharge pressure. The pump was considered inoperable and in accordance with Tech Spec requirements, reactor shutdown commenced. On July 21, sufficient flow was verified for ESW pump 52A and shutdown was terminated with plant load at 404 MWe.

On July 22, due to increasing containment spray heat exchanger differential pressure, the decision was made to proceed to cold shutdown to inspect and clean all containment spray heat exchangers, hydrolaze sections of ESW piping, flush the system and perform post-maintenance testing.

The plant remained in cold shutdown for the balance of the report period.

OPERATING DATA REPORT  
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: July 1985
3. UTILITY CONTACT: JOSEPH R. MOLNAR 609-971-4699
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe):  $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 650
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 620
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS: NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe): N/A
11. REASON FOR RESTRICTION, IF ANY: NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HRS	744.0	5088.0	136801.0
13. HOURS RX CRITICAL	495.3	4043.4	89671.3
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	469.7
15. HRS GENERATOR ON-LINE	486.4	3858.9	87395.6
16. UT RESERVE SHUTDOWN HRS	0.0	19.6	22.3
17. GROSS THERM ENER (MWH)	813200	6695140	143957469
18. GROSS ELEC ENER (MWH)	273330	2275190	48658185
19. NET ELEC ENER (MWH)	260541	2181479	46744044
20. UT SERVICE FACTOR	65.4	75.8	63.9
21. UT AVAIL FACTOR	65.4	76.2	63.9
22. UT CAP FACTOR (MDC NET)	56.5	69.2	55.1
23. UT CAP FACTOR (DER NET)	53.9	66.0	52.6
24. UT FORCED OUTAGE RATE	34.6	23.9	10.4
25. FORCED OUTAGE HRS	257.6	1209.5	10160.6
26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION): OCTOBER 1, 1985 - (1 Month)			
27. IF CURRENTLY SHUTDOWN ESTIMATED STARTUP TIME:		N/A	

AVERAGE DAILY POWER LEVEL  
NET MWe

DOCKET # . . . . . 50-219  
UNIT . . . . . Oyster Creek #1  
REPORT DATE . . . . . AUGUST 06, 1985  
COMPILED BY . . . . . WILLIAM J. EMRICH, JR.  
TELEPHONE # . . . . . 609-971-4637

MONTH JULY, 1985

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	623	16.	549
2.	621	17.	583
3.	618	18.	597
4.	616	19.	603
5.	612	20.	604
6.	610	21.	448
7.	609	22.	264
8.	74	23.	0
9.	78	24.	0
10.	481	25.	0
11.	474	26.	0
12.	487	27.	0
13.	412	28.	0
14.	474	29.	0
15.	477	30.	0
		31.	0

REFUELING INFORMATION - July, 1985

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: April 12, 1986

Scheduled date for restart following refueling: October 13, 1986

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Yes

Scheduled date(s) for submitting proposed licensing action and supporting information:

December, 1985

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - fuel design and performance analysis methods have been approved by the NRC. New operating procedures, if necessary, will be submitted at a later date.
2. Exxon Fuel Assemblies - no major changes have been made nor are there any anticipated.

The number of fuel assemblies (a) in the core	=	560
(b) in the spent fuel storage pool	=	1204
(c) in dry storage	=	4

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present licensed capacity: 2,600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Reracking of the fuel pool is in progress. Four out of ten (10) racks have been installed to date. When reracking is completed, discharge capacity to the spent fuel pool will be available until 1990 refueling outage.

## Monthly Operating Report

The following Licensee Event Reports were submitted during the month of July 1985:

Licensee Event Report 50-219/85-012 - On June 12, 1985, a mechanical failure of the plant's Electric Pressure Regulator (EPR) resulted in a decrease in reactor pressure. This pressure drop caused automatic closure of the Main Steam Isolation Valves, resulting in an automatic reactor scram. Although all rods inserted to shutdown the reactor, one of the two Scram Discharge Volumes (SDVs) did not fully isolate. The resulting flow of hot water through the volume caused steam and paint fumes to be emitted in the Reactor Building. This in turn caused activation of the deluge fire system on the 51' elevation of the Reactor Building. The Isolation Condensers were not initially used for plant cooldown due to high reactor water level. While attempting to place the Reactor Water Cleanup System in service to lower reactor water level, a cleanup system isolation valve failed to open due to its breaker tripping. The Electromatic Relief Valves (EMRVs) were used to control pressure until reactor water level was reduced (by letdown via the Cleanup System) to the point where the Isolation Condensers could be used. Reactor pressure was then controlled through manual actuation of the Isolation Condensers and pressure was reduced to less than 600 psig. The scram signal was reset, and the plant was placed in the cold shutdown condition. Repairs to the EPR, the two SDV valves, and the cleanup system valve were completed prior to plant startup following this event.

Licensee Event Report 50-219/85-013 - On Saturday, June 29, 1985 at approximately 0020 hours, Drywell to Torus differential pressure was found to be below the minimum value of 1.0 psid as specified in Technical Specifications. At 1130 hours on June 26, 1985, the pen which records torus pressure on Control Room recorder 12XR6 was turned off. When the pen was returned to service (at 0020 hours on June 29, 1985), the Torus to Drywell differential pressure was 0.89 psid. Within 10 minutes after discovery, the required Torus to Drywell differential pressure was restored by purging the Torus.



**GPU Nuclear Corporation**

Post Office Box 388  
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609 971-4000  
Writer's Direct Dial Number:

August 14, 1985

Director  
Office of Management Information  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Mr. Drew Holland at (609) 971-4643.

Very truly yours,

Peter B. Fiedler  
Vice President and Director  
Oyster Creek

PBF:KB:dsm(0170A)  
Enclosures

cc: Director (10)  
Office of Inspection and Enforcement  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

Dr. Thomas E. Murley, Administrator  
Region I  
U.S. Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

NRC Resident Inspector  
Oyster Creek Nuclear Generating Station

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