

TABLE OF CONTENTS

I. Operational Summary	<u>Page</u>
Performance Summary	1
Significant Operational Events	1-3
Fuel Performance and Spent Fuel Storage Capabilities	3
PORCs and Safety Valves Summary	3
Licensee Events, Special Reports and	3
Diesel Generator Failure Reports	4
Offsite Dose Calculation Manual Changes	4
Changes, Tests, and Experiments Not Requiring	5
Authorization From NRC Pursuant to 10 CFR 50.59 (a)	
II. Operating Statistics	
A. NRC Reports	
Operating Data Reports	6-7
Unit Shutdowns and Power Reductions	8-9
Average Daily Unit Power Level	10-11
B. TVA Reports	
Nuclear Plant Operating Statistics	12
Unit Outage and Availability	13-14
Reactor Histograms	15-16
III. Maintenance Summary	
Electrical Maintenance	17-19
Instrument Maintenance	20-23
Mechanical Maintenance	24
Modifications	25-29

Operations Summary

July 1985

The following summary describes the significant operational activities for the month of July. In support of this summary, a chronological log of significant events is included in this report.

Unit 1

Unit 1 was critical for 691.9 hours, produced 742,300 MWH (gross), resulting in an hourly gross load of 1,106,919 kW during the month. The capacity factor for the month was 84.3 percent. There are 46.21 full power days estimated remaining until the end of cycle 3 fuel. The cycle 3 refueling/modification outage is scheduled to begin September 27, 1985.

During the month the unit experienced one reactor scram, one power reduction, and no manual shutdowns.

Unit 2

Unit 2 was critical for 744 hours, produced 852,220 MWH (gross), resulting in an average hourly gross load of 1,145,457 kW during the month. The capacity factor for the month was 96.8 percent. There are 173.49 full power days estimated remaining until the end of cycle 3 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached February 21, 1986.

During the month, the unit experienced no reactor scrams, manual shutdowns or power reductions.

Significant Operational Events

Unit 1

<u>Date</u>	<u>Time</u>	<u>Event</u>
07/01/85	0001	The reactor was in mode 1 at 100% power, producing 1150 MWe.
07/05/85	1705	MSR C-1 was removed from service. The turbine dropped 6.0 MWe.
07/19/85	0349	The reactor tripped on a lo-lo level in #3 steam generator. An ASE was swapping the feeds to Turbine MOV Board B when the main oil pump tripped causing the high and low pressure stop valves to the main feedpump turbine to close. The auxiliary oil pump started but failed to produce the required pressure to keep the valves open. The unit operator inserted the rods to adjust for feedwater fluctuation but no runback occurred. There was a manual runback, MFPT A came up to full speed and the motor operated AFW pumps started. The generator was at 850 MWE when the reactor tripped.

Significant Operational Events (Cont.)

	<u>Unit 1</u>	
<u>Date</u>	<u>Time</u>	<u>Event</u>
07/21/85	0756	The reactor was taken critical.
	0914	The reactor was at 3% power and in a chemistry hold.
07/22/85	0100	Began power ascension.
	0157	The reactor entered mode 1.
	0315	The reactor was at 12% power and rolling turbine.
	0435	The turbine tripped on first out electrical problem- - too much excitation.
07/22/85	0524	The unit was tied on-line and increasing power.
	0600	The reactor was at 26% power and holding while the feedwater heater were being placed inservice.
	0720	Began power ascension.
	1040	The reactor obtained 30% power and entered a chemistry hold.
	2125	Began power ascension.
07/23/85	0625	Reactor was at 43% power and preparing to pump forward.
	0750	#3 Htr Dr Tk Pmps A & B tripped.
	0852	#3 Htr Dr Tk Pmps B & C tripped.
	0940	1-FCV-6-106 maintaining level in #3 Htr Dr Tk. Began pumping forward.
	0945	Began Power Ascension
07/24/85	0115	The reactor obtained 100% power.
07/26/85	0358	1-LCV-6-105B opened. The turbine was ranback to 67% GV position. The reactor was at 78% power.
	0446	The controlling setpoints were adjusted on #3 HDT. Began power ascension.
07/26/85	0930	The reactor obtained 100% power.
07/31/85	2359	The reactor was in mode 1 at 100% power producing 1145 MWe. The unit has been in continous operation since July 22, 1985 at 0524C (9 days).

Significant Operational Events (Cont.)

Unit 2

<u>Date</u>	<u>Time</u>	<u>Event</u>
07/01/85	0001	The reactor was in mode 1 at 100% power producing 1150 MWe.
07/31/85	2359	The reactor was in mode 1 at 100% power producing 1145 MWe. The unit has been in continuous operation since May 24, 1985 at 0141C (68 days).

Fuel Performance

Unit 1

The core average fuel exposure accumulated during July was 1033.83 MWD/MTU with the total accumulated core average fuel exposure of 12418.71 MWD/MTU.

Unit 2

The core average fuel exposure accumulated during July was 1189.46 MWD/MTU with the total accumulated core average fuel exposure of 7303.08 MWD/MTU.

SPECIAL NUCLEAR MATERIAL SAFEGUARDS & ACCOUNTABILITY SYSTEM ANNUAL AUDIT

The annual review of the plant's special nuclear material (SNM) safeguards and accountability system was performed on July 30-31, 1985. In addition to the records audit, a physical inventory was performed on July 31 in the new fuel storage vault on new fuel, 72 bundles, (Unit 1 Cycle 4). There were no discrepancies.

Spent Fuel Pit Storage Capabilities

The total storage capability in the spent fuel pit (SFP) is 1386. However, there are five cell locations which are not capable of storing spent fuel. Four locations (A10, A11, A24, A25) are unavailable due to a suction strainer conflict and one location (A16) is unavailable due to an instrumentation conflict. Presently, there is a total of 276 spent fuel bundles stored in the SFP. Thus, the remaining storage capacity is 1105.

PORVs and Safety Valves Summary

No PORVs or safety valves were challenged in July 1985.

Licensee Events and Special Reports

The following licensee event reports (LER) were sent during July 1985 to the Nuclear Regulatory Commission.

LER

DESCRIPTION OF EVENT

1-85025	With unit 1 in mode 5 and unit 2 in mode 1 at 100% reactor power, the service building radiation monitor 0-RM-90-132 was declared inoperable on May 28, 1985 at 1010C. The plant technical specifications require a noble gas sample to be taken every eight hours for up to 30 days or suspend all releases. The required sample at 1810C on June 1, 1985, was not taken until 2055C.
---------	--

Licensee Events and Special Reports

LER

DESCRIPTION OF EVENT (Cont.)

- 1-85026 On June 11, 1985, at 0130C with unit 1 in mode 4 at 270°F a main feedwater (MFW) isolation occurred. The isolation was initiated from a high high steam generator (S/G) level in steam generator #1. Makeup water to the generators was being supplied from the auxiliary feedwater system and the main steam isolation valve (MSIV) bypass valves were opened to heat the steam lines downstream of the MSIVs. The warming continued until the indicated $\Delta P > 25$ psid across the MSIVs was obtained. When the MSIV to S/G #1 was opened, the water level indication increased from 35 percent to 100 percent. The feedwater isolation signal is generated by any 2 of the 3 level indicators on any one generator being greater than or equal to 75 percent. Thus the feedwater isolation occurred.
- 1-85027 On June 25, 1985, at 0914C with unit 1 in mode 3 at 525°F a main steam line isolation occurred. The isolation resulted in steam generators loops 2 and 4 coincident with a operating temperature below 540°F (Lo-Lo Tavg). The isolation valves went to the closed position as required. The safety injection (SI) signal that is initiated by high steam line flow coincident with Lo-Lo Tavg was blocked, as allowed by STS; therefore, no SI occurred.
- 1-85028 At 0800C on July 4, 1985, with unit 1 in mode 1 at 100% power, the hourly fire watch for the unit 1 additional equipment building was not conducted because door A-183 would not open. The defective door was returned to operable status and the hourly fire watch resumed at 0906C.
- 2-85008 On July 12, 1985, at 1234C with unit 1 and unit 2 both in mode 1 at 100 percent power, an inadvertent trip of the normal feeder breaker on the 6900 volt 2A-A shutdown board initiated the actuation of the engineers safety feature (ESF) system. The trip occurred while a modification was being performed on the alternate feeder breaker. The alternate feeder breaker was racked out and personnel were not actually performing work immediately before or during the trip. There was no actual blackout condition during the event.

Special Reports

No special reports were transmitted during the month.

Diesel Generator Failure Report

There were no diesel generator failure reports transmitted during the month.

Offsite Dose Calculation Manual Changes

No changes were made to the Sequoyah Offsite Dose Calculation Manual during the month.

Changes, Tests and Experiments Not Requiring Authorization From The NRC
Pursuant to 10 CFR 50.59(a).

A. Change Description

The condensate demineralizer waste evaporator is part of the liquid radwaste processing system. Chapter 11 of the Final Safety Analysis Report is currently being revised to agree with the current "as-configured" system description. This change is part of this documentation.

The current configuration of the liquid radwaste system in brief, is to use the CDWE as the primary method for processing station leakage (floor drains). Portable demineralizers are available onsite to provide backup for the CDWE in case of its failure or excess leakage (over and above the CDWE capacity).

B. Unreviewed Safety Question Determination

1. The addition of the CDWE adds a permanent enhancement to the overall plant capability to process liquid radwaste. The CDWE is used continuously and thereby provides a constant demand upon the component cooling system which provides cooling water to the CDWE. The CDWE requires approximately 1600 gpm at full flow conditions and is capable of operation at lower capacities under situations of high CCS loading. The component cooling system, with the CCS pump in operation, can produce 6,000 gpm (SQN FSAR Section 9.2.1.2) per unit, to train B safety related loads. The capacity of the component cooling water system is adequate to assure that the demands of the CDWE during its normal operation, even though continuous, does not impact the cooling capability for engineered safeguard loads. Sufficient precautions are provided in the system, including automatic isolation of CCS to the CDWE, to assure that a break in the system does not divert extra flow to the CDWE portion of the system away from the safeguard loads.
2. The CDWE as a part of the liquid radwaste system is a portion of a system that could be part of a long-term recovery from a design basis accident. However, this part is not an integral part and is not essential to safe shutdown of the unit.

The interface of the CDWE with the CCS as a cooling medium is addressed in the discussion of No. 1 above. Since the system (CCS) is automatically isolable and provides adequate measures to manually isolate flow in the case of failure of this automatic function, the evaluation is that the system as currently configured and operated poses no additional possibility for creation of a new accident.

3. The Component Cooling System (CCS) interfaces with the CDWE as its primary cooling medium. Further the CCS is included in Technical Specifications requiring operability of two trains during power operation. The design features of the system providing isolation and the assurance that engineered safeguards loads are not deprived during neither normal or accident conditions proves assurance that the margin of safety defined in CCS specification is not decreased.

The CDWE provides additional, permanent, enhancement of the liquid radwaste system therefore the margin of safety of this system is increased.

OPERATING DATA REPORT

DOCKET NO. 50-327
DATE AUGUST 7 1985
COMPLETED BY GENE R WILBOURN
TELEPHONE (615) 870-6544

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1
2. REPORT PERIOD: JULY 1985
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: _____

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): _____

10. REASONS FOR RESTRICTIONS, IF ANY: _____

NOTES:

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.00	5087.00	35808.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	691.90	3269.25	23916.91
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	670.60	3234.30	23343.25
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	2206077.60	10603301.96	75280987.91
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	742300.00	3643040.00	25379456.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	716172.00	3494071.00	24375701.00
19. UNIT SERVICE FACTOR	90.13	63.58	65.19
20. UNIT AVAILABILITY FACTOR	90.13	63.58	65.19
21. UNIT CAPACITY FACTOR (USING MDC NET)	83.85	59.83	59.30
22. UNIT CAPACITY FACTOR (USING DER NET)	83.85	59.83	59.30
23. UNIT FORCED OUTAGE RATE	9.87	12.44	17.57
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Refueling/Modification - September 27, 1985 - 51 days (Cycle 3)			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			

NOTE THAT THE THE YR.-TO-DATE AND
CUMULATIVE VALUES HAVE BEEN UPDATED.

OPERATING DATA REPORT

DOCKET NO. 50-328
DATE AUGUST 7, 1985
COMPLETED BY GENE R WILBOURN
TELEPHONE (615) 870-6544

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2
2. REPORT PERIOD: JULY 1985
3. LICENSED THERMAL POWER(MWT): 3411.0
4. NAMEPLATE RATING (GROSS MWE): 1220.6
5. DESIGN ELECTRICAL RATING (NET MWE): 1148.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBERS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:-----

NOTES:

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):-----

10. REASONS FOR RESTRICTIONS, IF ANY:-----

	THIS MONTH	YR.-TO-DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.00	5087.00	27768.00
12. NUMBER OF HOURS REACTOR WAS CRITICAL	744.00	4789.22	21484.34
13. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14. HOURS GENERATOR ON-LINE	744.00	4724.17	20994.35
15. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16. GROSS THERMAL ENERGY GENERATED (MWH)	2535400.80	15435603.15	67434614.02
17. GROSS ELECTRICAL ENERGY GEN. (MWH)	852669.00	5277519.00	22969199.00
18. NET ELECTRICAL ENERGY GENERATED (MWH)	822669.00	5077706.00	22098714.60
19. UNIT SERVICE FACTOR	100.00	92.87	75.61
20. UNIT AVAILABILITY FACTOR	100.00	92.87	75.61
21. UNIT CAPACITY FACTOR (USING MDC NET)	96.32	86.95	69.32
22. UNIT CAPACITY FACTOR (USING DER NET)	96.32	86.95	69.32
23. UNIT FORCED OUTAGE RATE	0.00	7.02	8.24
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Refueling/Modification - February 1986 - 50 days (Cycle 3)-----			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:-----			

NOTE THAT THE THE YR.-TO-DATE AND
CUMULATIVE VALUES HAVE BEEN UPDATED.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327
 UNIT NAME Sequoyah One
 DATE August 5, 1985
 COMPLETED BY T. J. Hollomon
 TELEPHONE (615) 870-7132

REPORT MONTH JULY

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
4	850719	F	73.58	A	3				Lo-Lo Level #3 Steam Generator
5	850726	F	5.53	A	5				

1

F: Forced
 S: Scheduled

2

Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3

Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Cont. of Existing
 Outage
 5-Reduction
 9-Other

4

Exhibit G-Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

5

Exhibit I-Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-328
UNIT NAME Sequoyah Two
DATE August 5, 1985
COMPLETED BY T. J. Hollomon
TELEPHONE (615) 870-7132

REPORT MONTH JULY

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
						NONE			

1

F: Forced
S: Scheduled

2

Reason:

A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3

Method:

1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Cont. of Existing
Outage
5-Reduction
9-Other

4

Exhibit G-Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

5

Exhibit I-Same Source

(9/77)

SQNP
AI-18
Appendix A
Page 3 of 8
Rev. 24

ATTACHMENT 1
AVERAGE DAILY UNIT POWER LEVEL

FILE PACKAGE NO. 55
REPORT REQUIREMENTS

DOCKET NO. 50-327
UNIT One
DATE August 6, 1985
COMPLETED BY T. J. Holloman
TELEPHONE (615) 870-7132

MONTH July 1985

Day	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>1113</u>	17	<u>1107</u>
2	<u>1113</u>	18	<u>1104</u>
3	<u>1117</u>	19	<u>1050</u>
4	<u>1119</u>	20	<u>NA</u>
5	<u>1115</u>	21	<u>NA</u>
6	<u>1119</u>	22	<u>227</u>
7	<u>1120</u>	23	<u>659.6</u>
8	<u>1121</u>	24	<u>1097</u>
9	<u>1120</u>	25	<u>1103</u>
10	<u>1118</u>	26	<u>1075</u>
11	<u>1119</u>	27	<u>1104</u>
12	<u>1117</u>	28	<u>1104</u>
13	<u>1115</u>	29	<u>1109</u>
14	<u>1115</u>	30	<u>1107</u>
15	<u>1114</u>	31	<u>1105</u>
16	<u>1112</u>		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

*SQNP
AI-18
Appendix A
Page 3 of 8
Rev. 24

ATTACHMENT 1
AVERAGE DAILY UNIT POWER LEVEL

FILE PACKAGE NO. 55
REPORT REQUIREMENTS

DOCKET NO. 50-328
UNIT Two
DATE August 6, 1985
COMPLETED BY T. J. Holloman
TELEPHONE (615) 870-7132

MONTH JULY 1985

Day AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>1107</u>
2	<u>1107</u>
3	<u>1106</u>
4	<u>1108</u>
5	<u>1107</u>
6	<u>1107</u>
7	<u>1110</u>
8	<u>1111</u>
9	<u>1112</u>
10	<u>1112</u>
11	<u>1108</u>
12	<u>1114</u>
13	<u>1113</u>
14	<u>1112</u>
15	<u>1110</u>
16	<u>1107</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>1105</u>
18	<u>1104</u>
19	<u>1101</u>
20	<u>1104</u>
21	<u>1103</u>
22	<u>1103</u>
23	<u>1102</u>
24	<u>1102</u>
25	<u>1103</u>
26	<u>1104</u>
27	<u>1105</u>
28	<u>1106</u>
29	<u>1106</u>
30	<u>1111</u>
31	<u>1110</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

TVA 6560C (PP-3-76)

UNIT OUTAGE AND AVAILABILITY

SEQUOYAH

Nuclear Plant

Licensed Reactor Power 3411 MW(th)

Unit No. ONE

Generator Rating 1220.5 MW(e)

Month/Year JULY 1985

Design Gross Electrical Rating 1183 MW

Period Hours 744

Day	Time Unit Available						Time Not Available								Unit				OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION
	Total		Gen.		Not Used		Turbine		Gen.		Reactor		Unit		Time Out		Time In					
	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min				
1	24	00	24	00			00	00	00	00	00	00	00	00								
2	24	00	24	00			00	00	00	00	00	00	00	00								
3	24	00	24	00			00	00	00	00	00	00	00	00								
4	24	00	24	00			00	00	00	00	00	00	00	00								
5	24	00	24	00			00	00	00	00	00	00	00	00								
6	24	00	24	00			00	00	00	00	00	00	00	00								
7	24	00	24	00			00	00	00	00	00	00	00	00								
8	24	00	24	00			00	00	00	00	00	00	00	00								
9	24	00	24	00			00	00	00	00	00	00	00	00								
10	24	00	24	00			00	00	00	00	00	00	00	00								
11	24	00	24	00			00	00	00	00	00	00	00	00								
12	24	00	24	00			00	00	00	00	00	00	00	00								
13	24	00	24	00			00	00	00	00	00	00	00	00								
14	24	00	24	00			00	00	00	00	00	00	00	00								
15	24	00	24	00			00	00	00	00	00	00	00	00								
16	24	00	24	00			00	00	00	00	00	00	00	00								
17	24	00	24	00			00	00	00	00	00	00	00	00								
18	24	00	24	00			00	00	00	00	00	00	00	00								
19	03	49	24	00			00	00	00	00	20	11	20	11	03	49		Lo-Lo level #3 Steam Generator	Auto Scram	Mode 3		
20	00	00	24	00			00	00	00	00	24	00	24	00								
21	00	00	24	00			00	00	00	00	07	56	24	00								
22	18	36	24	00			00	00	00	00	00	00	05	24		05	24					
23	24	00	24	00			00	00	00	00	00	00	00	00								
24	24	00	24	00			00	00	00	00	00	00	00	00								
25	24	00	24	00			00	00	00	00	00	00	00	00								
26	24	00	24	00			00	00	00	00	00	00	00	00								
27	24	00	24	00			00	00	00	00	00	00	00	00								
28	24	00	24	00			00	00	00	00	00	00	00	00								
29	24	00	24	00			00	00	00	00	00	00	00	00								
30	24	00	24	00			00	00	00	00	00	00	00	00								
31	24	00	24	00			00	00	00	00	00	00	00	00								
Total	670	25	744	00			00	00	00	00	52	07	73	35								

TVA 6560C (PP-3-76)

UNIT OUTAGE AND AVAILABILITY

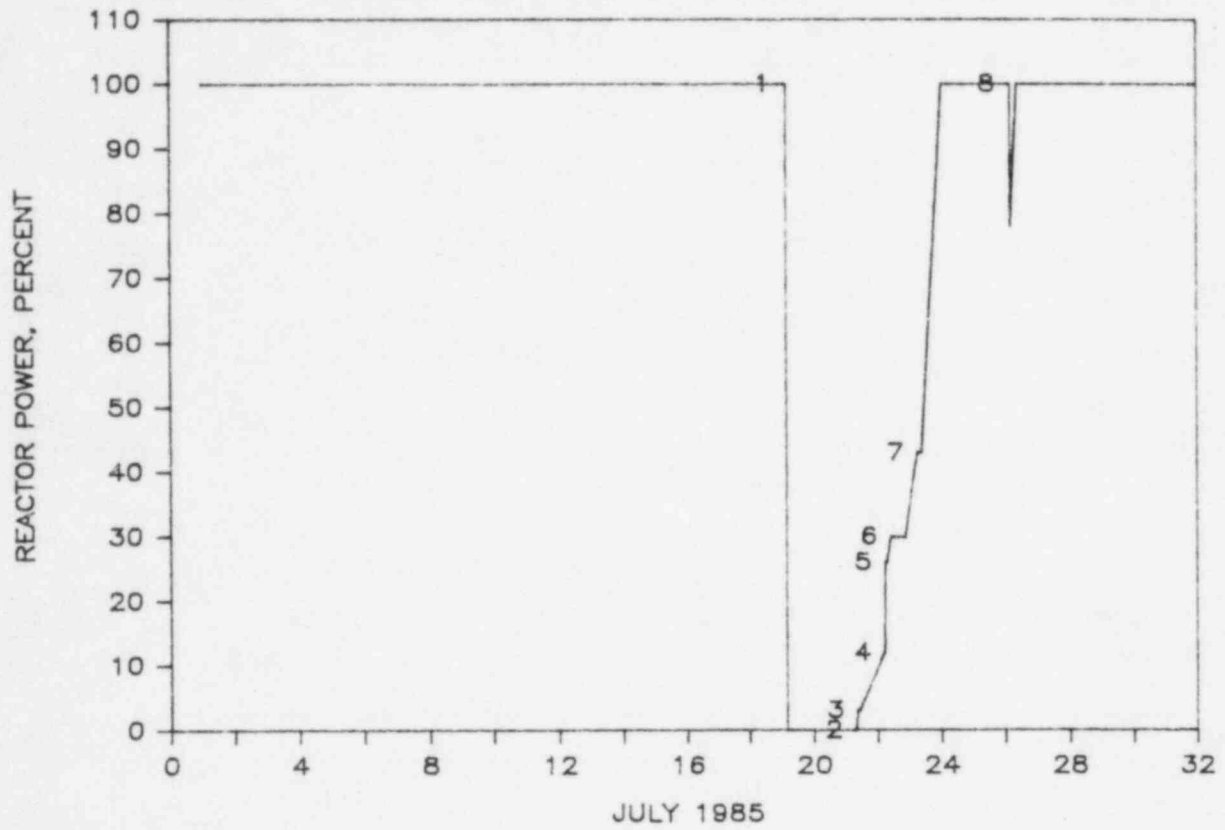
SEQUOYAH

Nuclear Plant

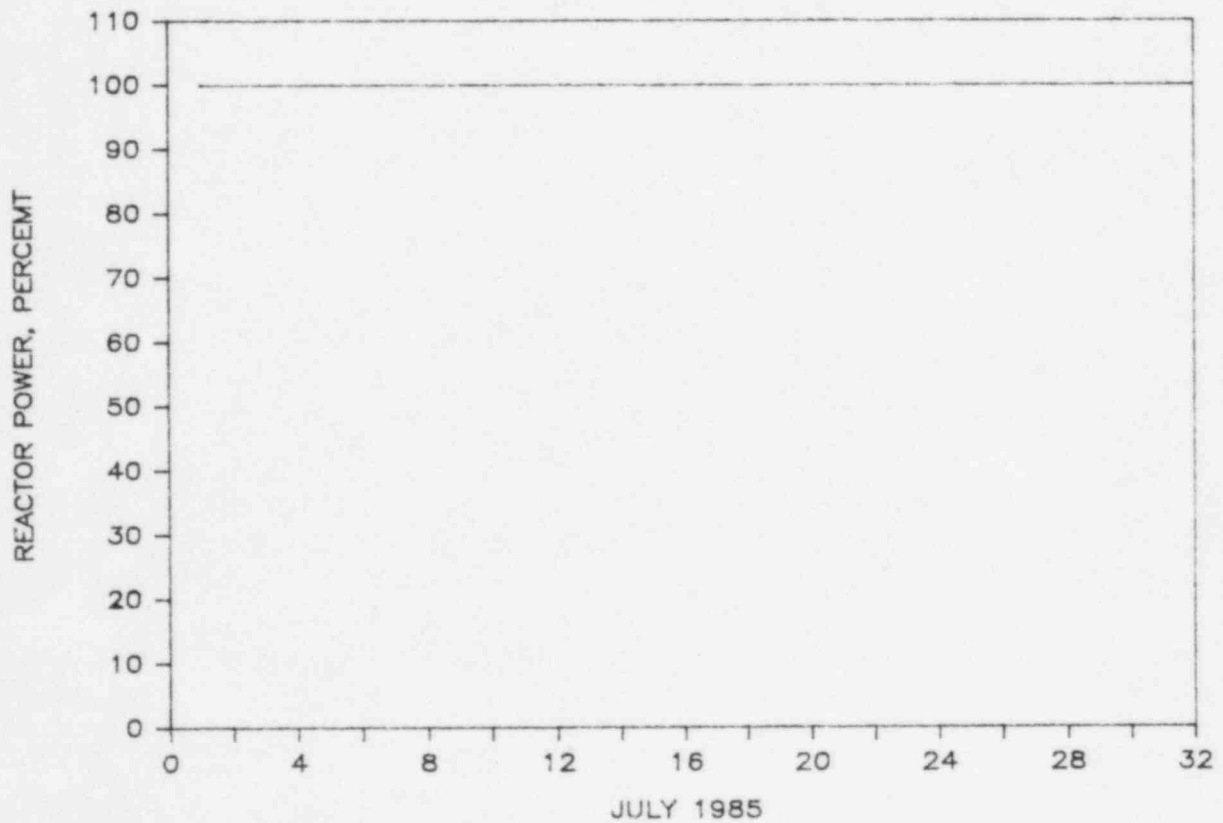
Licensed Reactor Power 3411 MW(th)Unit No. TWOGenerator Rating 1220.5 MW(e)Month/Year JULY 1985Design Gross Electrical Rating 1183 MWPeriod Hours 744

Day	Time Unit Available						Time Not Available								Unit				OUTAGE CAUSE	METHOD OF SHUTTING DOWN REACTOR	UNIT STATUS DURING OUTAGE	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION
	Total		Gen.		Not Used		Turbine		Gen.		Reactor		Unit		Time Out		Time In					
	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min	Hrs	Min				
1	24	00	24	00			00	00	00	00	00	00	00	00								
2	24	00	24	00			00	00	00	00	00	00	00	00								
3	24	00	24	00			00	00	00	00	00	00	00	00								
4	24	00	24	00			00	00	00	00	00	00	00	00								
5	24	00	24	00			00	00	00	00	00	00	00	00								
6	24	00	24	00			00	00	00	00	00	00	00	00								
7	24	00	24	00			00	00	00	00	00	00	00	00								
8	24	00	24	00			00	00	00	00	00	00	00	00								
9	24	00	24	00			00	00	00	00	00	00	00	00								
10	24	00	24	00			00	00	00	00	00	00	00	00								
11	24	00	24	00			00	00	00	00	00	00	00	00								
12	24	00	24	00			00	00	00	00	00	00	00	00								
13	24	00	24	00			00	00	00	00	00	00	00	00								
14	24	00	24	00			00	00	00	00	00	00	00	00								
15	24	00	24	00			00	00	00	00	00	00	00	00								
16	24	00	24	00			00	00	00	00	00	00	00	00								
17	24	00	24	00			00	00	00	00	00	00	00	00								
18	24	00	24	00			00	00	00	00	00	00	00	00								
19	24	00	24	00			00	00	00	00	00	00	00	00								
20	24	00	24	00			00	00	00	00	00	00	00	00								
21	24	00	24	00			00	00	00	00	00	00	00	00								
22	24	00	24	00			00	00	00	00	00	00	00	00								
23	24	00	24	00			00	00	00	00	00	00	00	00								
24	24	00	24	00			00	00	00	00	00	00	00	00								
25	24	00	24	00			00	00	00	00	00	00	00	00								
26	24	00	24	00			00	00	00	00	00	00	00	00								
27	24	00	24	00			00	00	00	00	00	00	00	00								
28	24	00	24	00			00	00	00	00	00	00	00	00								
29	24	00	24	00			00	00	00	00	00	00	00	00								
30	24	00	24	00			00	00	00	00	00	00	00	00								
31	24	00	24	00			00	00	00	00	00	00	00	00								
Total	744	00	744	00			00	00	00	00	00	00	00	00								

SEQUOYAH ONE REACTOR HISTOGRAM



SEQUOYAH TWO REACTOR HISTOGRAM



Reactor Histogram Comments

July 1985

Unit 1

1.	07/19/85	0350	Reactor Tripped - Lo-Lo level #3 steam generator.
2.	07/21/85	0755	The reactor was taken critical
3.	07/21/85	0907	Chemistry Hold
4.	07/22/85	0531	Tied on-line
5.	07/22/85	0600	Placed feedwater heaters inservice
6.	07/22/85	0726	Chemistry Hold
7.	07/23/85	0628	#3 HTR Dr TK Pmps tripped
8.	07/26/85	0405	Manual runback when 1-LCV-6-105B opened.

14:08:29 DATE....	08-02-85 COMPONENT.....	ELECTRICAL MAINTENANCE MONTHLY REPORT FOR JULY			PAGE 1
		FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
06-18-85	1-BCTD-313-JE/ 110-A	AIR CONDITIONER WOULD NOT RUN	OVERLOAD RELAY BURNED OUT	REPLACED OVERLOAD RELAY	A518125
06-19-85	1-FCV-062-0073	VALVE INDICATING LIGHTS SHOWING INCORRECT VALVE POSITION	ACTUATOR BOLT WAS LOOSE PREVENTING LIMITS FROM MAKING UP	REINSTALLED ACTUATOR BOLT AND CHECKED FOR PROPER OPERATION	A528961
06-20-85	1-TS-13-200A&B	TROUBLE ALARM IS IN ON SMOKE DETECTOR	BAD DETECTOR	REPLACED DETECTOR	A528425
06-20-85	2-BCTC-77-125A	PUMP WILL RUN FOR 30 SEC. THEN TRIP ON THERMAL OVERLOAD	BAD AUXILIARY CONTACT ON BREAKER	REPLACED AUXILIARY CONTACTS ON BREAKER	A526388
06-21-85	0-CHGB-250-QE- D	CHARGER KEEPING 142 VOLTS ON VITAL BATTERY BOARD	CHARGER OUT OF ADJUSTMENT CAUSING FALSE VOLTAGE READING	ADJUSTED FLOAT VOLTAGE AND EQUALIZE VOLTAGE	A531041
06-21-85	1-FCV-30-19	PURGE AIR DAMPER INDICATOR LIGHT IS SHOWING INCORRECT VALVE POSITION	LIMIT SWITCH BRACKET WAS LOOSE	ADJUSTED AND TIGHTENED LIMIT SWITCH BRACKETS	A528964
06-24-85	2-HTCK-234-106	POINT 10 CIRCUIT 106 THERMOSTAT WILL NOT OPERATE	VIBRATION CAUSED LOOSE THERMOSTAT CONNECTION	REPAIRED LOOSE CONNECTION	A528213
06-29-85	2-HTCK-63-54	TEMPERATURE LESS THAN TECH SPEC REQUIREMENT	BAD CONTROLLER	REPLACED CONTROLLER	A528448
07-10-85	1-LSV-62-118A	VALVE POSITION INDICATOR SHOWING INCORRECT VALVE POSITION	ACTUATOR ARM LOOSE ON STEM CAUSING FALSE READING	ADJUSTED ACTUATOR ARM AND CHECKED FOR PROPER OPERATION	A528292
07-10-85	2-INVB-250-QP- E	FAN FAILURE ALARM IN WHILE FAN IS RUNNING	DIRT IN SWITCH	CLEANED AIR FLOW SWITCH AND CHECKED FOR PROPER	A528416

14:08:29 DATE....	08-02-85 COMPONENT.....	ELECTRICAL MAINTENANCE MONTHLY REPORT FOR JULY			PAGE 2
		FAILURE DESCRIPTION.....	CAUSE OF FAILURE.....	CORRECTIVE ACTION.....	MR.NO..
				OPERATION	
07-10-85	1-PMLA-68-341F	CONTROL BANK OF PRESSURIZER HEATERS WAS NOT MODULATING TO CONTROL PRESSURE	BAD SILICON CONTROL RECTIFIER	REPLACED SILICON CONTROL RECTIFIER	A527920
07-10-85	1-LSV-062-0118 A	VALVE INDICATING LIGHTS SHOWING INCORRECT POSITION	ACTUATION ARM OUT OF ADJUSTMENT	ADJUSTED ACTUATION ARM AND CHECKED FOR PROPER OPERATION	A528292
07-15-85	2-FCV-63-063	BOTH INDICATOR LIGHTS ARE ON WHEN VALVE IS IN OPEN OR CLOSE POSITION	ACTUATOR ARM SHIFTED ON STEM	ADJUSTED LIMIT SWITCH ACTUATOR ARM	A526450
07-15-85	2-FSV-63-21	ELECTRICAL SOLENOID THAT OPERATES VALVE IS NOT FIRMLY ATTACHED CAUSING ERRATIC VALVE OPERATION	MISSING RETAINER CAP	REPLACED RETAINER CAP	A528053
07-15-85	2-FCV-063-0063	VALVE INDICATOR LIGHTS WILL NOT CLEAR WHEN VALVE IS OPENED OR CLOSED	LIMIT SWITCH ACTUATOR ARM OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH ACTUATOR ARM AND CHECKED FOR PROPER OPERATION	A526450
07-16-85	2-MTR8-313-JL/ 5E-B	CHILLER WILL RUN 5-10 SEC. THEN SHUT OFF	AIR FLOW SWITCH OUT OF ADJUSTMENT	ADJUSTED AIR FLOW SWITCH	A541052
07-19-85	2-FCV-063-0084	VALVE INDICATOR LIGHTS SHOWING INCORRECT VALVE POSITION	VALVE ACTUATOR ARM OUT OF ADJUSTMENT	ADJUSTED VALVE ACTUATOR ARM	A531388
07-21-85	1-FCV-001-0025	VALVE WILL NOT OPEN	BROKEN HAND SWITCH	REPLACED ROTARY SWITCH	A533531
07-24-85	1-BKRA-030-015 7	BREAKER BLOWS CONTROL POWER FUSES	CRYDOM RELAY ON DAMPER WAS FAULTY	REPLACED RELAY AND CHECKED FOR PROPER OPERATION	A537148

14:08:29 08-02-85 ELECTRICAL MAINTENANCE MONTHLY REPORT FOR JULY PAGE 3
 DATE.... COMPONENT..... FAILURE DESCRIPTION..... CAUSE OF FAILURE..... CORRECTIVE ACTION..... MR.NO..

DATE	COMPONENT	FAILURE DESCRIPTION	CAUSE OF FAILURE	CORRECTIVE ACTION	MR.NO.
07-25-85	1-BCTB-030-003 9	REPAIR 480V ACB FOR 1B-B CONTAINMENT AIR RETURN FAN	SHORTED CRYDOM RELAY	REPLACED CRYDOM RELAY IN CONTROL CIRCUIT	A537135

20 records listed.

INSTRUMENT MAINTENANCE

Unit 1

1. During full power operation, while operations was switching a 480V AC MOV board supply, 1B main oil pump lost power. A subsequent decrease in oil pressure caused the main feed pump turbine stop valves to close but did not cause a feed pump turbine trip. Power was immediately returned to the oil pump causing an increase in pressure which did not allow the turbine to runback or to initiate a start of the auxiliary feedwater pumps. Without manual actuation the stop valves would not reopen. With only A pump operating a lo lo level occurred in the #3 steam generator which caused an ESF actuated reactor trip.
2. During the forced outage, following maintenance on FCV-3-103, the #4 steam generator level increased sharply to 75% when the valve was unisolated which resulted in a feedwater isolation. An inspection of the valve indicated that the valve was not seated. The valve stroke was readjusted and the valve was returned to service.
3. During startup from the ice outage the RVLIS PMT showed that the control room indicators did not respond as expected. A calibration check was performed on the sections of the RVLIS electronics that were expected to be causing the error. This check showed that some cards had drifted from their calibrated values. The recalibration of the affected cards was completed.
4. Performed a calibration of UHI level switches, SI-196. Two switches, LS-87-21 and 23, were found out of technical specification tolerance. PRO 1-85-228 was written.

Unit 2

1. The initial calibration of RVLIS has been completed. Installation of the system was completed during U2C2 outage.
2. Performed a calibration of UHI level switches, SI-196.2. One switch, LS-87-23, was replaced because of a stripped setpoint screw.
3. The process computer malfunctioned eight times. The problem was determined to be an intermittent amplifier on a read head for the disk. This caused the P250 to stop and wait for data, which never arrived. Troubleshooting was difficult because no error messages were generated.

COMP

MR.	COMP	U	FUNC	SYS	ADDRESS.	DATE....	DESCRIPTION.....	CORRECTIVE ACTION.....
A237837	2	FS		030	195	07/25/85	2-FS-030-195, REPAIR AS REQ'D FLOW CONTROLS OF PEN RM CLR FAN 288	PROBLEM WITH 2 FS 30 194. WROTE UP TO HAVE ABOVE SW CALIB
A292790	2	TS		068	3/6	07/19/85	2-TS-068-3/6, CHECK CALIB OF 2TS 68 316 RECALIB IF REQ'D	BISTABLE OUT OF CAL. RECAL BISTABLE
A292792	2	TS		068	317	07/19/85	2-TS-068-317, REQ'DK CALIB OF 2TS 68 317 RECALIB IF REQ'D	BISTABLE OUT OF CAL. RECAL BISTABLE
A527102	2	PIS		087	23	07/15/85	2-PIS-087-23, INDICATOR NOT WORKING PROPERLY	PRESS IND INOP. REPLACED IND-RECALIB.
A527109	2	FT		070	81E	07/24/85	2-FT-070-81E, CHECK CALIB OF FT IT IS CAUSING PERIODIC ISOL OF ISOL VLVS FCV 70 90 AND 133	XMTR WAS OUT OF CALIB. RECALIB XMTR.
A527116	2	TI		074	38C	07/25/85	2-TI-074-38C, *I* TEMP IND NEEDS TO BE CALIB NEEDED TO COMPLETE SI 3 MONTHLY DUE BY 7/29/85	RESIDUAL HEAT REMOVAL SYS OUTLET HEAT EXCH 'A' TEMP CURRENT TO CURRENT REPEATER WAS FOUND SLIGHTLY OUT OF CALIB. MODIFIER WAS RECALIB.
A527904	1	LIC		003	172	07/03/85	1-LIC-003-172, LCV-3-172 IS TAKING TOO LONG TO INDICATE FULLY CLOSED	NO PROBLEM FOUND. OPER. CHECK AND STROKE OK.
A528245	1	FT		070	149	07/11/85	1-FT-070-149, PERFORM LOOP CALIB CK ON FT 70 149	FLOW SW WAS OUT OF CALIB. RECALIB SW.
A528246	1	FT		070	150	07/11/85	1-FT-070-150, PERFORM LOOP CK CALIB ON FT 70 150	XMTR AND FLOW SW OUT OF CALIB. RECALIB XMTR AND FLOW SW
A528247	0	FT		070	20	07/11/85	0-FT-070-20, PERFORM LOOP CK CALIB ON FT 70 20	XMTR OUT OF CAL. RECAL XMTR
A528250	2	LI		003	173	07/08/85	2-LI-003-173, LI INDICATING SLIGHTLY LOWER THAN OTHER #2 S/G LEVEL INDICATING SOURCES.	CURRENT TO CURRENT REPEATER HAD A BAD CAPACITOR IN IT. REPLACED BAD CAPACITOR; RECAL; AND RETURNED TO SERVICE
A528952	1	MR		030	241	07/03/85	1-MR-030-241, RECORDER HAS FAILED LOW	BAD DEW CELL. REPLACED DEW CELL
A531376	2	TE		070	154	07/11/85	2-TE-070-154, PERFORM LOOP CALIB ON TE 70 154	TEMP MODIFIER WAS OUT OF CALIB. RECALIB THE TEMP MODIFIER
A531378	2	FT		070	150	07/10/85	2-FT-070-150, PERFORM LOOP CK CALIB ON FT 70 150	FLOW MODIFIER WAS OUT OF CALIB. RECALIB THE FLOW MODIFIER
A531379	2	FT		070	149	07/10/85	2-FT-070-149, PERFORM LOOP CHECK CALIB ON FT 70 149	FLOW MODIFIER WAS OUT OF CALIB. RECALIB. FLOW MODIFIER
A531382	2	TS		068	316	07/19/85	2-TS-068-316, TS-68 IS KEEPING ALARM PZR SPARY LINE TEMP ON IN WHEN INDICATOR READWS 550 DEGREES F	BISTABLE OUT OF CAL. RECAL PER MR 292790
A533588	1	FI		003	103B	07/22/85	1-FI-003-103B, LOOP #4 FEED FLOW CHANNEL DOES NOT INDICATE FLOW. INVESTIGATE AND REPAIR. PLACE BISTABLES IN TRIPPED CONDITION PER INST.	FDWTR FLOW XMTR WAS OUT OF CALIB. RECALIB XMTR
A533639	0	PS		082	271	07/26/85	0-PS-082-271, PS WILL NOT START AC ON LOW RECEIVER PRESS	LACK OF LUBRICATION. LUBRICATED AND CALIB SW
A533698	2			085	88	07/17/85	2--085-88, *I* RPI SEEMS TO HAVE A LOOSE CONNECTION	LOOSE CONNECTION. REPAIRED PLUG ON IND.
A537115	1	LI		063	49	07/06/85	1-LI-063-49, LVL IND SHOWS LOWER THAN ACTUAL	AIR IN LINES AND HEAT TRACE TOO HIGH. BLEW DOWN LINES AND RESET HEAT TRACE TO

COMP

MR.	COMP	U	FUNC	SYS	ADDRESS	DATE	DESCRIPTION	CORRECTIVE ACTION
A538647	1	PIS	001	13		07/23/85	1-PIS-001-13, RECAL SW	PROPER POINT. PRESS. SW OUT OF CAL. RECAL SW.
A541822	0	SC	046	1830645		07/03/85	0-SC-046-1830645, REPLACE AMPLIFIER GAIN POTENTIOMETER. CHECK OUT SPEED CONTROL MODULE FOR PROPER OPERATION AND RETURN TO POWERSTORES	BAD POTENTIOMETER. REPLACED POT AND RETURNED TO PWR STORES FOR SPARE
A541823	0	SC	046	1929411		07/03/85	0-SC-046-1929411, CHECK MODULE FOR PROPER OPERATION AND RETURN TO PSTORES FOR SPARE	NONE. CHECKED PROPER OPERATION AND SENT BACK TO PSTORES
A543789	1	FS	030	103		07/03/85	1-FS-030-103, VERIFIED 1A SUPPLY FAN AUX BLDG IS RUNNING WITH ALL DAMPERS OPEN	BAD SW. REPLACED SW AND RECAL
A545843	2	LS	087	22		07/10/85	2-LS-087-22, NEED TO REPLACE SW WITH ANOTHER SW BECAUSE ADJUSTMENT SCREW IS STRIPPED ON OLD SW	SCREW STRIPPED OUT ON SW. REPLACED SW.

25 records listed.

COMP

MR.HIST U FUNC SYS ADDRESS. DATE.... DESCRIPTION..... CORRECTIVE ACTION.....

MR.HIST U	FUNC	SYS ADDRESS	DATE	DESCRIPTION	CORRECTIVE ACTION
A292778	1 LT	068 320	07/01/85	1-LT-068-320, PLACE A RECORDER ON THE THREE PRZR/LVL CHANNELS 1 LT 68 320; 335; 339 CONNECT THE RECORDER TO THE COMPUTER POINTS IN 1 R 26 AND 27	NONE TESTING. HOOK UP RECORDER TO 3 PZR LVL CHANNELS FOR TESTING PURPOSES

One record listed.

Mechanical Maintenance Section

July 1985

Unit 0

- 1) Performed the monthly inspection on the diesel generators.
- 2) Performed the five year inspection on 2B-B diesel generator.
- 3) Performed the six month inspection on 1A-A diesel generator.
- 4) Installed a threshold on an ABSCE boundary, door A118, to repair a two inch gap.
- 5) Rebuilt the "B" waste gas compressor.
- 6) Rebuilt the fuel oil pump on the 2A-A diesel generator.
- 7) Rebuilt the "B" auxiliary air compressor.

Unit 1

- 1) Welded a nipple and valve onto the drain of the 1B2 moisture separator reheater.
- 2) 1-FCV-63-84 was leaking through. Replaced the plug, the stem, and the packing.
- 3) Tightened the expansion joint on A&B main feedwater pump turbines to lower air inleakage.
- 4) Replaced the main turbine bearing oil lift pump and relief valve.

Unit 2

- 1) Rebuilt 2-VLV-67-582D relief valve.
- 2) Repaired a 3/4" schedule 40 letdown sample line which was broken at the heat affected zone of a joint weld. The joint was replaced with a 4" section of piping.

SUMMARY OF WORK COMPLETED

MODIFICATIONS

JULY 1985

NUREG 0588

ECN 5970 - MOV Operator Replacement

The last operator was replaced this period.

Appendix R

ECN 5435 - Install Weatherstripping on ABSCE Doors

We finished installing weatherstripping on ABSCE architectural doors. We are awaiting material for heavy equipment doors.

ECN 6235 - Reroute Various Cables

Work continues with five workplans in work.

ECN 6305 - Elevation 714 Fire Barrier

We have finished installing baseplates and prefabricating structural steel framing for the wall.

ECN 6308 - Bypass Switch

This ECN has been completed.

ECN 6309 - Breaker Indicator Lights

This ECN has been completed.

ECN 6319 - Fire Protection Piping

We have rerouted approximately 30 percent of the heads required to be relocated. Work is ongoing to plug sprinkler heads that are to be abandoned (approximately 70 percent complete). We are adding heads on elevations 714, 734, and 749 of the Auxiliary Building. The workplan to install two new deluge valve systems has been written and is in the approval cycle.

Other Items

ECNs 2783 and 5202 - Fifth Diesel Generator

Permanent power tie-ins have been made. The system is waiting for removal of temporary feeds prior to being energized. Work continues on providing permanent power to the building. A punchlist has been prepared for remaining work items and cleanup.

Other Items (Continued)

ECN 5009 - ERCW Piping Changeout from Carbon Steel to Stainless Steel

Work continues on train A.

ECN 5111 - Semipermanent Power to Temporary Facilities

This project has begun. Seven handholes have been installed. Approximately 35 percent of the conduit in areas A, B, and C is complete.

ECN 5111 - High-Pressure Fire Protection to OE Complex

Piping is complete, and hydrostatic testing will be performed within a few days.

ECNs 5111 and 5503 - Office and Power Stores Facility

Power Stores began moving into the new facility on July 15. The modification to the NRC office on elevation 710 has been completed. Work has begun on the installation of the windows in the stairwells. Work is continuing to complete the remaining punchlist items. Most of the outstanding work consists of exterior items, such as sidewalks, handrails, manhole covers, and landscaping. The modification to the vending area of elevation 710 will begin within the next two weeks.

ECN 5119 - Install Radiation Monitor Cables in Conduit

The last monitors have been completed. This ECN is complete.

ECN 5194 - Iodine Monitor

The two security doors have been functionally tested. This ECN is complete.

ECN 5200 - Postaccident Sampling Facility

Rework of postmodification test deficiencies is on hold for the final design of one level loop.

ECN 5237 - Laundry Facility

Remaining work is on hold until the last wall is built.

ECN 5373 - Condensate Demineralizer Air Compressor

This project is almost complete, with the exception of the installation of the motor coupling and pressure control instrument. The materials are onsite. Functional testing and minor repairs will be done with the help of a vendor representative during August. We received a new differential pressure control box to replace a faulty one found on the new compressor. Once Construction installs the control box, the new air compressor can be functionally tested by the vendor representative.

Other Items (Continued)

ECN 5489 - Parking Lot No. 1

This project has begun and is underway at the present time. The work area was blocked off, and a temporary access to the parking lot was made. Grading is in process for both additions to the parking lot.

ECN 5599 - Fifth Vital Battery

The Appendix R-related conduit wrap is approximately 95 percent complete. The wrapping will be completed when additional materials are available. The protective coating in the battery room will be applied during August.

ECNs 5609 and 5610 - Makeup Water Treatment Plant

This project is near completion. A walkdown was performed on July 29. The participants in the walkdown indicated items which need to be completed, and a punchlist has been compiled. The septic tank and piping have arrived, and digging has begun for their installation. Still awaiting design procurement are the stainless steel sump pump and the air compressor.

ECN 5613 - Installation of Emergency Lights

Work has been completed on eight of nine lighting systems. The last one is being held because of defective equipment.

ECN 5620 - Add Instrumentation for Auxiliary Feedwater Pump

Conduit work has commenced.

ECN 5657 - Installation of MSR Drain Valves

The unit 2 workplan to install approximately 150 drain valves is in the approval cycle. Insulation reinstallation on unit 1 still remains incomplete.

ECN 5660 - Blocking Diodes

This modification is approximately 50 percent complete.

ECN 5664 - Replace Relays in Wells Fargo Alarms

Remaining work has been restarted and will continue during Power Block reconfiguration.

ECN 5795 - Field Services Building

Fire detection system work is in hold for materials.

Other Items (Continued)

ECN 5841 - Hot Machine Shop

This project has been completed as designed under ECN 5841. The addition of a monorail and electro-polisher in the decontamination room will be accomplished under ECN 6411.

ECN 5855 - Replace Air Lock Doors A56 and A57

We have finished chipping the concrete column between doors and have installed the air tank and control panel. The workplan to install door A56 is in the approval cycle, and the one for door A57 is ready for approval.

ECN 5865 - Relocate Level Alarm LA-77-129

Preoutage work is in process.

ECN 5878 - Modify Entrainment Separator - CDWE

This work is essentially complete.

ECNs 5932, 5935, 5959, and 6105 - Security Power Block

All cameras and watch towers have been completed. One fence sector, three gates, and three doors remain incomplete at this time. The E-field for the ERCW has been completed as well. The anti-intrusion devices around pipes in the Turbine Building have been installed. Other remaining work includes CCW pump E-field, high mast lighting, painting of ERCW coffer/cells, and high intensity lighting for the ERCW pumping station.

ECN 5938 - Feedwater Heater Replacement

The three monorails have been installed and tested. Work continued on fabrication and installation of steel platforms required to transport the Nos. 1 and 2 feedwater heaters to elevation 685 of the Turbine Building. Some building windbraces were removed to allow feedwater heater movement to begin. Tests were conducted on the internal cleanliness of the new Nos. 1 and 2 feedwater heaters, and they were found to be acceptable for installation without further cleaning.

ECN 5990 - Add Condensate Divert Valve to CDWE

This work is essentially complete.

ECN 6057 - Cable Tray Covers

Approximately 240 out of 290 cable tray covers have been remanufactured or replaced.

Other Items (Continued)

ECN 6204 - Electrical Penetration Overcurrent Protection

Fuse replacement and fuse block installation are complete. We are waiting for a Technical Specification change to place the circuits in operation.

ECN 6326 - Install Temporary Drain Hose from CDWE to Floor Drain Collector Tank

Work was completed on the rubber hose that was run from the CDWE to the floor drain collector tank.

ECN 6351 - Install Demineralized Water Booster Pumps for CDWE

Operations requested that the booster pump tie-in point be located further upstream on a demineralized water line to help boost the water pressure for the entire CDWE Building. This change required drawing revision and new material, which will add approximately five days to the original outage duration.

ECN 6362 - Install Pad Flanges and Valves for Sample Skid - CDWE

This work is essentially complete.

ECN 6417 - Install Alternate Seal Water for Pumps - CDWE

Mechanical drawings from OE were issued, and the workplan was written.

ECN 6447 - Incore Detector Ten-Path Cart Modification

The following modifications have been completed for units 1 and 2 to qualify the incore detector ten-path cart to withstand a seismic event: (1) Existing 3/8-inch bolts in ten-path cart anchorage were replaced with 3/8-inch ASME SA-193 grade B7 bolts. (2) Removable sections of platform grating in the cart area were mechanically secured. (3) Existing 12 bolts in ten-path frame assembly were replaced with 3/4-inch ASTM A325 bolts. In addition, frame members were increased in size. (4) For unit 2 only, additional braces for cart anchorage were installed.

Training Buildings

This project has been completed, and the buildings are being occupied at this time.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
P. O. Box 2000
Soddy-Daisy, Tennessee 37379

August 13, 1985

Nuclear Regulatory Commission
Office of Management Information
and Program Control
Washington, DC 20555

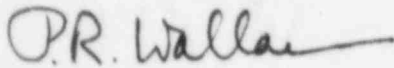
Gentlemen:

SEQUOYAH NUCLEAR PLANT - MONTHLY OPERATING REPORT - JULY 1985

Enclosed is the July 1985 Monthly Operating Report to the NRC for
Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace
Plant Manager

Enclosure

cc (Enclosure):

Director, Region II
Nuclear Regulatory Commission
Office of Inspection and Enforcement
101 Marietta Street
Suite 3100
Atlanta, GA 30323 (1 copy)

Director, Office of Inspection
and Enforcement
Nuclear Regulatory Commission
Washington, DC 20555 (10 copies)

Mr. A. Rubio, Director
Electric Power Research Institute
P. O. Box 10412
Palo Alto, CA 94304 (1 copy)

Mr. R. C. Goodspeed
MNC 461
Westinghouse Electric Corporation
P. O. Box 355
Pittsburgh, PA 15230 (1 copy)

Mr. K. M. Jenison, Resident NRC Inspector
Sequoyah Nuclear Plant

Director, Office of Management
Information and Program Control
Nuclear Regulatory Commission
Washington, DC 20555 (2 copies)

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339 (1 copy)

1983-TVA 50TH ANNIVERSARY

An Equal Opportunity Employer

TE24
11