

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

JULY 1985

At the onset of the July reporting period, power generation continued at 98% Reactor Rated Thermal Power (47 MWe-net). At 0850 on July 10, the 1A Forced Circulation Pump Discharge Valve closed reducing power from 97% to 65%. Power was then manually reduced below 50% per Technical Specification. The valve also closed twice on July 11, each time reducing power from approximately 45-50% to 25-30%. Valve closure was postulated to be due to spiking on the loop differential temperature unit output and "sluggish" operation of the automatic valve control system.

On July 14 a reactor shutdown was commenced from approximately 96% power due to a small steam leak at No. 3 Feedwater Heater. The leak was temporarily patched on July 15. Power was reduced to approximately 40% prior to re-escalation.

Operating power was reached on July 17. Power generation ranged from 95-98% from July 17-25. At 1115 on July 25, the reactor automatically shutdown due to a ground in the low gas pressure scram circuit on Control Rod Drive Mechanism No. 8. While shutdown, Fuse 55/2, which is in the rod control circuit, blew. The fuse was replaced. The grounded wire was located and repaired. The reactor was returned to critical at 0538 on July 26. During the startup, at approximately 0.06% power, a high flux spike occurred at 1051 on Nuclear Instrumentation Channel 5 when its range switch was upscaled. The spike could not be duplicated. The range switch was cleaned. During the shutdown the power range Nuclear Instrumentation recorder was replaced.

The reactor was taken critical at 1655 on July 26. The turbine-generator was connected to the DPC grid at 0426 on July 27. On July 30, 97% power was achieved. Power generation continued at 96-97% for the remainder of the reporting period.

During the month of July troubleshooting was conducted following failure of the 1A High Pressure Service Water/Alternate Core Spray Diesel. Refer to LER 85-13 for details.

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R PDR

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OPERATING DATA REPORT

DOCKET NO. 50-409
 DATE 08/06/85
 COMPLETED BY Loree Malin
 TELEPHONE 608-689-2331

OPERATING STATUS

1. Unit Name: La Crosse Boiling Water Reactor
2. Reporting Period: 0000, 07/01/85 to 2400, 07/31/85
3. Licensed Thermal Power (MW_t): 165
4. Nameplate Rating (Gross MW_e): 65.3
5. Design Electrical Rating (Net MW_e): 50
6. Maximum Dependable Capacity (Gross MW_e): 50
7. Maximum Dependable Capacity (Net MW_e): 48
8. If Changes Occur in Capacity Ratings (Items 3 Through 7) Since Last Report, Give Reasons: _____

NOTES:

9. Power Level To Which Restricted, If Any (Net MW_e): _____
10. Reasons for Restrictions, If Any: _____

	<u>This Month</u>	<u>Yr.-to-Date</u>	<u>Cumulative</u>
11. Hours In Reporting Period	744	5,087	138,050
12. Number Of Hours Reactor Was Critical	719.6	4,154.3	92,478.0
13. Reactor Reserve Shutdown Hours	0	0	478
14. Hours Generator On-Line	702.8	4,038.7	85,942.0
15. Unit Reserve Shutdown Hours	0	0	79
16. Gross Thermal Energy Generated (MWH)	100,732.3	566,252.2	11,913,654.0
17. Gross Electrical Energy Generated (MWH)	30,299	174,213	3,569,824
18. Net Electrical Energy Generated (MWH)	28,386	163,089	3,308,925
19. Unit Service Factor	94.5	79.4	62.3
20. Unit Availability Factor	94.5	79.4	62.3
21. Unit Capacity Factor (Using MDC Net)	79.5	66.8	49.9
22. Unit Capacity Factor (Using DER Net)	76.3	64.1	47.9
23. Unit Forced Outage Rate	5.5	3.1	9.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: NA
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-409

UNIT LACBWR

DATE 08/06/85

COMPLETED BY L.S. Goodman

TELEPHONE 608-689-2331

MONTH July 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	<u>47</u>
2	<u>47</u>
3	<u>47</u>
4	<u>47</u>
5	<u>47</u>
6	<u>47</u>
7	<u>47</u>
8	<u>47</u>
9	<u>46</u>
10	<u>28</u>
11	<u>18</u>
12	<u>28</u>
13	<u>35</u>
14	<u>41</u>
15	<u>22</u>
16	<u>34</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	<u>44</u>
18	<u>46</u>
19	<u>46</u>
20	<u>46</u>
21	<u>46</u>
22	<u>46</u>
23	<u>46</u>
24	<u>46</u>
25	<u>21</u>
26	<u>0</u>
27	<u>12</u>
28	<u>27</u>
29	<u>38</u>
30	<u>45</u>
31	<u>46</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH July 1985DOCKET NO. 50-409UNIT NAME LACBWRDATE 08/06/85COMPLETED BY L.S. GoodmanTELEPHONE 608-689-2331

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
85-05	07/10/85	F	0	A	4	NA	CB	INSTRU	Power decreased from 97% to 65% when the 1A Forced Circulation Pump Discharge Valve closed when a spurious high differential loop temperature signal occurred. Power was manually reduced below 50%.
85-06	07/14/85	F	0	A	4	NA	HH	PIPEXX	Manual shutdown was commenced due to a small steam leak at No. 3 Feedwater Heater. The leak was temporarily patched and power was re-escalated.
85-07	07/25/85	F	41.2	A	3	85-14	RB	ELECON	Reactor shutdown automatically due to a ground in the gas pressure scram circuit on Control Rod Drive Mechanism No. 8. Troubleshooting was conducted which determined

1

F: Forced
S: Scheduled

2

Reason:

A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3

Method:

1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4

Exhibit G-Instructionse
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File
(NUREG-0161)

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Exhibit 1 - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-409
 UNIT NAME LACBWR
 DATE 08/06/85
 COMPLETED BY L.S. Goodman
 TELEPHONE 608-689-2331

REPORT MONTH July 1985

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
85-06 (CONTINUED)	07/25/85	F	41.2	A	3	85-14	RB	ELECCN	location of the ground. Wire was repaired. The range switch for Nuclear Instrumentation Channel 5 was cleaned following a scram during startup which occurred when the switch was upscaled. The recorder for the power range Nuclear Instrumentation channels was replaced during the shutdown.

1

F: Forced
S: Scheduled

2

Reason:

A-Equipment Failure (Explain)
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3

Method:

1-Manual
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 4-Other (Explain)

4

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 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG-0161)

5

Exhibit 1 - Same Source

August 6, 1985

In reply, please
refer to LAC-11062

DOCKET NO. 50-409

Office of Management Information
and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

SUBJECT: DAIRYLAND POWER COOPERATIVE
LA CROSSE BOILING WATER REACTOR (LACBWR)
PROVISIONAL OPERATING LICENSE NO. DPR-45
MONTHLY OPERATING DATA REPORT FOR JULY 1985

Reference: (1) NRC Letter, Reid to Madgett,
dated September 19, 1977.
(2) NRC Letter, Reid to Madgett,
dated December 29, 1977

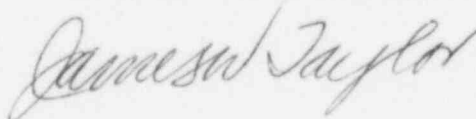
Gentlemen:

In accordance with instructions contained in Reference (1) and Technical Specification Amendments contained in Reference (2), we are submitting information concerning operability and availability of the La Crosse Boiling Water Reactor (LACBWR) for the Month of July 1985.

Please contact us if there are any questions concerning this report.

Very truly yours,

DAIRYLAND POWER COOPERATIVE



James W. Taylor, General Manager

JWT:LSG:sks

cc: J. G. Keppler, Regional Administrator, NRC-DRO III
Walter Paulson, LACBWR Project Manager
NRC Resident Inspector
D. Sherman (ANI Library)
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