

NRC MONTHLY OPERATING REPORT

DOCKET NO. 50-528
UNIT NAME PVNGS-1
DATE 8/8/85
COMPLETED BY M. P. Richardson
TELEPHONE 602-932-5300
Ext. 6593

OPERATING STATUS

1. Unit Name: Palo Verde Nuclear Generating Station, Unit 1
2. Reporting Period: July 1985
3. Licensed Thermal Power (MWt): 3800
4. Nameplate Rating (Gross MWe): 1304
5. Design Electrical Rating (Net MWe): 1270
6. Maximum Dependable Capacity (Gross MWe): To be determined
7. Maximum Dependable Capacity (Net MWe): To be determined
8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____
N/A
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	5088	5088
12. Number Of Hours Reactor Was Critical	397	1092	1092
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	293	668	668
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	488786.9	817607.7	817607.7
17. Gross Electrical Energy Generated (MWH)	135,000	195,200	195,200
18. Net Electrical Energy Generated (MWH)	95,869	126,661	126,661
19. Unit Service Factor	N/A	N/A	N/A
20. Unit Availability Factor	N/A	N/A	N/A
21. Unit Capacity Factor (Using MDC Net)	N/A	N/A	N/A
22. Unit Capacity Factor (Using DER Net)	N/A	N/A	N/A
23. Unit Forced Outage Rate	N/A	N/A	N/A
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shutdown At End Of Report Period, Estimated Date of Startup: 8/27/85

26. Units In Test Status (Prior To Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	5/85	5/25/85
INITIAL ELECTRICITY	6/85	6/10/85
COMMERCIAL OPERATION	11/85	N/A

AVERAGE DAILY UNIT POWER LEVEL

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MONTH: July 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	228
2	0
3	195
4	3
5	36
6	293
7	405
8	151
9	352
10	397
11	207
12	0
13	0
14	0
15	30
16	335

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	266
18	0
19	148
20	397
21	402
22	151
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	0

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

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- 7/1 0000 - Reactor at 40.11% power.
- 7/1 1815 - Reactor tripped on hi pressurizer pressure resulting from loss of main feed pump "A" on low suction pressure.
- 7/2 1500 - Commenced CEA withdrawal to criticality, Reactor is critical.
- 7/2 1537 - Reactor power at 10-4%, plant in Mode 3.
- 7/3 0547 - Began power ascension to above 20%.
- 7/3 1939 - Reactor power at 35% generator at 358 MWe.
- 7/4 0058 - Shutdown main turbine and placed on turning gear per GE recommendation due to sustained hi vibration on several bearings following load reduction from 35% to 11% Reactor power.
- 7/5 1240 - Paralleled Main Gen. w/grid, picked up load to ~40 MW.
- 7/6 0910 - Stabilized Reactor power ~42% power and turbine load ~ 455 MWe.
- 7/6 1138 - Reactor power at 50% power, 535 MWe.
- 7/8 0850 - EHC leak on #1 control valve, started reducing turbine load and Reactor power.
- 7/8 0930 - Tripped main turbine
- 7/8 0945 - Reactor power approximately 13.5%
- 7/8 1835 - EHC leak resolved, reset Main Turbine
- 7/8 2130 - Main Turbine at 1800 RPM.
- 7/9 2325 - Reactor at 19.1% power
- 7/9 0219 - Increasing Reactor power at 10%/hr to 50% power.
- 7/9 0519 - Reactor power at 50%.

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- 7/11 0400 - Entered Action Statement concerning RCS leak.
- 7/11 1124 - Commenced reducing turbine load
- 7/11 1308 - Tripped Main Turbine
- 7/11 1331 - Entered Mode 3
- 7/12 RCS in cooldown at 400F, 1950 PSIA; all CEAs fully inserted.
- 7/12 1730 - Stopped RCS cooldown, stabilized plant at 360F-370F to repressurize for RCS leak diagnosis.
- 7/13 Plant in Mode 3, four hour RCS leak rate test in process.
- 7/13 0848 - Abnormal RCS leak rate, troubleshooting to find source of leak.
- 7/13 NRC granted a 72 hour extension for RCS leakage.
- 7/14 1552 - Exited Action Statement for unidentified RCS leakage greater than 1gpm.
- 7/14 Informed NRC via ENS that we were exiting 72 hr extension granted for RCS leakage.
- 7/14 2200 - Reactor critical
- 7/15 1635 - Plant in Mode 2 at 2% power.
- 7/15 1720 - Reactor power >5%; entered Mode 1.
- 7/15 1818 - Main Turbine at 1800 rpm.
- 7/15 2217 - Reactor power raised to ~ 25%
- 7/16 0615 - Started power ascension to 50%
- 7/16 1015 - Leveled Reactor power at 50%.
- 7/17 1540 - Reactor trip. Trip due to a hardware failure in CEAC 2 which resulted in random erroneous penalty factors being sent to the CPCs for DNBR and LPD.
- 7/18 0400 - CEAC 2 declared operable following replacement of a faulty memory board and satisfactory retest.
- 7/18 2105 - Commenced Reactor startup.
- 7/18 2210 - Reactor critical, entered Mode 2.
- 7/18 2250 - Increased Reactor power to 2%.

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- 7/19 0407 - Entered Mode 1
- 7/19 0756 - Received main turbine trip on high exhaust hood temperature.
- 7/19 0816 - Reset main turbine.
- 7/19 1152 - Synchronized main generator, commenced increase to 18% power.
- 7/19 1730 - Reactor power at 50%
- 7/22 0748 - Shutdown Circulating Water Pumps C&D for suspected leakage of circulating water into condensate system.
- 7/22 0830 - Commenced reducing reactor power
- 7/22 1016 - Maintaining Reactor power at ~3%.
- 7/23 0534 - Started Reactor shutdown
- 7/23 0550 - Broke vacuum in condenser.
- 7/23 Investigating cause of high UNIDENTIFIED LEAKAGE from RCS
- 7/23 Reactor trip breakers open, heat removal on Atmospheric Dump Valves, nonessential AFW for S/G feed, draining condenser hotwells.
- 7/24 0143 - Entered Mode 4.
- 7/24 0654 - Entered Mode 5
- 7/24 0705 - Stabilizing RCS temp at 200-205F.
- 7/25 1243 - Isolated "A" charging pump suction. RCS leakage from VCT stopped. Observed water leaking back into the seal lube tank when the suction was re-opened. Leak rate approx. 1 gpm. This is postulated to be the cause of the ~1 gpm unidentified RCS leakage.

UNIT SHUTDOWNS AND POWER REDUCTIONS

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 TELEPHONE 932-5300
 Ext 6593

No.	Date	Type	Duration (Hours)	Reason	Method of Shutting Down Reactor	LER No.	System Code	Component Code	Cause & Corrective Action to Prevent Recurrence
2	7/01/85	F	22	A	3	1-85-043-00	HBG	N/A	Small mesh suction strainers on FW pumps used for startup were blocked causing a high pressure drop. Proper strainers were installed.
3	7/04/85	F	27	G	5				Due to excessive rate of cooldown there was a high vibration on several bearings on the MT Weld was leaking. Rewelded electrohydraulic #1 control valve.
4	7/08/85	F	10	A	1				
5	7/11/85	F	125	A	1	1-85-042-00	PCF	Pump	Unidentified RCS leakrate over Tech Spec limit during performance of procedure. Procedure reperformed for different time intervals leakrate was then within Tech Spec limits.
6	7/17/85	F	43	A	3	1-85-049-00	IBD	INTCPM	Faulty memory board on CEAC. Replaced memory board.
7	7/22/85	F	232	A	1				Leak was from charging pump packing. Replaced packing on charging pump.

1
 F-Forced
 S-Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation from
 Previous Month
 5-Reduction of 20%
 or Greater in the
 Past 24 Hours
 6-Other (Explain)

4
 Exhibit F - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG 0161)
 5
 Exhibit H-Same Source

REFUELING INFORMATION

DOCKET NO. 50-528

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COMPLETED BY M. P. Richardson

TELEPHONE 602-932-5300

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1. Scheduled date for next refueling shutdown.

03/01/87

2. Scheduled date for restart following refueling.

04/19/87

3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Not Yet Determined
What will these be?

Not Yet Determined

4. Scheduled date for submitting proposed licensing action and supporting information.

Not Yet Determined

5. Important Licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Not Yet Determined

6. The number of fuel assemblies.

a) In the core. 241

b) In the spent fuel storage pool. 0

7. Licensed spent fuel storage capacity. 1329

Intended change in spent fuel storage capacity. None

8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.

2002 (w/annual reloads and full core discharge capability).



Arizona Nuclear Power Project

P.O. BOX 52034 • PHOENIX, ARIZONA 85072-2034

ANPP-33220-EEVB/GEC

August 15, 1985

Learned W. Barry, Director
Office of Resource Management
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Palo Verde Nuclear Generating Station (PVNGS)
Unit 1
Docket No. STN 50-528, License No. NPF-41
July Monthly Operating Report
File: 85-056-026; G.1.01.10

Dear Mr. Barry:

Attached please find the July 1985 Monthly Operating Report prepared and submitted pursuant to Specification 6.9.1.6 of Appendix A (Technical Specifications) to the Palo Verde Nuclear Generating Station, Unit 1 Operating License. By copy of this letter, we are also forwarding a copy of the Monthly Operating Report to the Regional Administrator of the Region V Office.

If you have any questions or concerns, please contact me.

Very truly yours,

E. E. Van Brunt, Jr.
Executive Vice President
Project Director

EEVBjr/GEC/slh
Attachments

cc: J. B. Martin
R. P. Zimmerman
E. A. Licitra
A. C. Gehr

EE2A
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