

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Callaway Plant Unit 1						DOCKET NUMBER (2) 0 5 0 0 0 4 8 3								PAGE (3) 1 OF 0 2													
TITLE (4) Intermediate Range Hi Flux Reactor Trip																											
EVENT DATE (5)				LER NUMBER (6)				REPORT DATE (7)				OTHER FACILITIES INVOLVED (8)															
MONTH	DAY	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		MONTH	DAY	YEAR		FACILITY NAMES Indian Point - 3				DOCKET NUMBER(S) 0 5 0 0 0 2 8 6											
0	5	0	6	8	5	-	0	2	5	-	0	1	0	9	2	7	8	5	0	5	0	0	0				
OPERATING MODE (9) 2				THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																							
				20.402(b)				20.405(c)				<input checked="" type="checkbox"/> X				50.73(a)(2)(iv)				73.71(b)							
POWER LEVEL (10) 0 0 0				20.405(a)(1)(i)				50.38(c)(1)								50.73(a)(2)(v)				73.71(c)							
				20.405(a)(1)(ii)				50.38(c)(2)								50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text NRC Form 366A)							
				20.405(a)(1)(iii)				50.73(a)(2)(i)								50.73(a)(2)(vii)(A)											
				20.405(a)(1)(iv)				50.73(a)(2)(ii)								50.73(a)(2)(viii)(B)											
				20.405(a)(1)(v)				50.73(a)(2)(iii)								50.73(a)(2)(ix)											
LICENSEE CONTACT FOR THIS LER (12) NAME Charles D. Naslund, Superintendent I & C																		TELEPHONE NUMBER AREA CODE 3 1 4 6 7 6 - 8 5 0 0									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																											
CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPROS		CAUSE	SYSTEM	COMPONENT	MANUFAC TURER	REPORTABLE TO NPROS																	
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR											
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input type="checkbox"/> NO		1	0	3	1	8	5								

On 5/6/85 at 2115 CDT a Reactor Trip, Feedwater Isolation, and Auxiliary Feedwater Actuation occurred during a reactor startup. The plant was in Mode 2, Startup, at 0% power at the time of the event.

The Reactor Trip was initiated by a spurious Intermediate Range Hi Flux Signal that was caused by a blown fuse in the ex-core neutron monitoring channel SE-NI-36. The spurious Hi Flux signal also initiated an automatic transfer of charging pump suction from the Volume Control Tank to the Refueling Water Storage Tank. The fuse was replaced and by 0947 on 5/7/85 normal reactor startup was commenced.

Discussions with Westinghouse Electric Corporation have indicated that a possible leak path exists for an alternating current to be superimposed onto the direct current in the Log Amplifier. The resultant current flow could then be sufficient to blow the control power fuse. Until a permanent solution is identified a temporary procedure change has been made which will remove the test signal to the Log Amplifier during reactor startups.

There was no damage to plant equipment or release of radioactivity as a result of this incident. This event has in no way affected the health or safety of the public.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Callaway Plant Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 8 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	0 2 5	0 1	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 5/6/85 at approximately 2115 CDT a Reactor Trip, Feedwater Isolation, and Auxiliary Feedwater Actuation occurred during a plant startup following the Reactor Trip reported in LER 85-024-00 transmitted via ULNRC-1103, dated 5/31/85. The plant was in Mode 2, Startup, at 0% power at the time of the event. This event is classified as reportable per 10CFR50.73(a)(2)(iv).

The Reactor Trip was initiated by an Intermediate Range Hi Flux Signal that was caused by a blown 5 ampere control power fuse in the ex-core neutron monitoring channel SE-NI-36 (IEEE Standard 805-1983 system-IG). The spurious Hi Flux signal also initiated an automatic transfer of charging pump suction from the Volume Control Tank to the borated Refueling Water Storage Tank. The blown fuse was replaced and by 0947 on 5/7/85 normal reactor startup was commenced with the reactor becoming critical at 1045.

Union Electric Company is presently evaluating the cause of the event with Westinghouse Electric Corporation. A similar event was discovered to have occurred at the Indian Point - 3 plant, NRC Docket No. 50-286, through the INPO Operating Plant Experience Report OE 1308. An investigation into the exact cause of this event is continuing and the results of this investigation will be provided in a supplement to this report.

It is believed that the Test Calibrate Module is providing a leakage path for an alternating current through the Test Mode Switch to the Log Amplifier. The alternating current could then be superimposed onto a direct current resulting in a current flow sufficient to blow the control power fuse.

Until a permanent solution is identified a temporary change to operating procedure OTG-ZZ-00002, Reactor Startup, has been made which directs technicians to unplug the test signal to the Log Amplifier on both intermediate range channels during a reactor startup.

All safety systems responded as designed and there was no damage to plant equipment or release of radioactivity as a result of this event. In no way has this event effected the health or safety of the public.

Previous occurrences: None