

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) DIABLO CANYON, UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 2 1 7 1 5										PAGE (3) 1 OF 0 1 1																													
TITLE (4) LOW LOW STEAM GENERATOR LEVEL REACTOR TRIP																																																	
EVENT DATE (5) MONTH DAY YEAR 0 8 2 7 8 5										LER NUMBER (6) YEAR SEQUENTIAL NUMBER REVISION NUMBER 8 5 - 0 3 0 - 0 0 0										REPORT DATE (7) MONTH DAY YEAR 9 2 6 8 5										OTHER FACILITIES INVOLVED (8) FACILITY NAMES DOCKET NUMBER(S) 0 5 0 0 0 0 0 0																			
OPERATING MODE (9) 1										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																																							
POWER LEVEL (10) 1 0 0										20.402(b) 20.4.5(a)(1)(i) 20.406(a)(1)(ii) 20.406(a)(1)(iii) 20.406(a)(1)(iv) 20.406(a)(1)(v)										20.408(c) 50.36(e)(1) 50.36(e)(2) 50.73(a)(2)(i) 50.73(a)(2)(ii) 50.73(a)(2)(iii)										X 50.73(a)(2)(iv) 50.73(a)(2)(v) 50.73(a)(2)(vii) 50.73(a)(2)(viii)(A) 50.73(a)(2)(viii)(B) 50.73(a)(2)(ix)										73.71(b) 73.71(c) OTHER (Specify in Abstract below and in Text, NRC Form 306A)									
LICENSEE CONTACT FOR THIS LER (12) NAME DAVID P. SISK, REGULATORY COMPLIANCE ENGINEER																														TELEPHONE NUMBER AREA CODE 8 0 5 5 9 5 - 7 3 5 1																			
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC																				
X	SJ	HS	W 1 2 0	N																																													
SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) X NO																														EXPECTED SUBMISSION DATE (15) MONTH DAY YEAR																			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 2202 PDT, August 27, 1985, while in Mode 1 (power operation) Unit 1 experienced a reactor trip (Engineered Safety Feature-Reactor Protection System). All automatic systems responded as designed. The four-hour Significant Event Report (10 CFR 50.72) was made at 2214 PDT.

This event was caused by a loose termination on the Main Feedwater Pump 1-2 (SJ) (P) local manual trip button (SJ) (HS). The loose termination caused the pump to trip which was followed by a plant transient resulting in a SG 1-4 low low steam generator (SJ) (SG) level reactor trip.

The loose termination was tightened and the local manual trip connections for the remaining Unit 1 feedwater pump were inspected. Both Unit 2 feedwater pump control stations were checked and found to have no loose connections.

Since no cause for the loose connection could be determined and no problems with this termination had been experienced before, no additional corrective action was deemed necessary.

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PACIFIC GAS AND ELECTRIC COMPANY

PG&E

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JAMES D. SHIFFER
VICE PRESIDENT
NUCLEAR POWER GENERATION

September 26, 1985

PGandE Letter No.: DCL-85-314

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Re: Docket No. 50-275, OL-DPR-80
Diablo Canyon Unit 1
Licensee Event Report 85-030-00
Low Low Steam Generator Level Reactor Trip

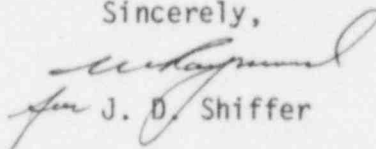
Gentlemen:

Pursuant to 10 CFR 50.73(a)(2)(iv), PGandE is submitting the enclosed Licensee Event Report concerning the inadvertent actuation of an Engineered Safety Feature (ESF), Reactor Trip.

This event has in no way affected the public's health and safety.

Kindly acknowledge receipt of this material on the enclosed copy of this letter and return it in the enclosed addressed envelope.

Sincerely,


for J. D. Shiffer

Enclosure

cc: L. J. Chandler
J. B. Martin

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