

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JULY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

8510010638 850731  
PDR ADOCK 05000373  
R PDR

## I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982, and commercial power operation was commenced on January 1, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

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II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

July 1-26

July 1, 0001 Hours	Reactor power at 6.3%.
July 1, 0230 Hours	Generator Synchronized to Grid
July 1, 0700 Hours	Reactor Power at 34%.
July 2, 1500 Hours	Reactor Power at 66%.
July 3, 2300 Hours	Reactor Power at 86%.
July 6, 0830 Hours	Suppression pool spray inoperable, commence 7 day timeclock.
July 11, 2300 Hours	Reactor Power at 47%.
July 12, 0615 Hours	Reactor manually scrammed. The reactor was critical for 270 hours and 15 minutes.

JULY 27-31

July 27, 1930 Hours	Reactor Critical
July 28, 1215 Hours	Generator Synchronized to Grid.
July 28, 2300 Hours	Reactor Power at 45%.
July 29, 0700 Hours	Reactor Power at 72%.
July 31, 2300 Hours	Reactor Power at 96%. The reactor was critical for 100 hours and 30 minutes. Totaling 370 hours and 45 minutes for the month of July.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no amendments to the facility license or Technical Specifications during this reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L35149	RHR Shutdown Cooling Discharge Valve, 1E12-F053A.	Eroded seat and disc.	Vave leaked in excess of allow- able limits.	Lapped seat and ground disc.
L49899	Hathaway Sequence-of- Events recorder.	Faulty series switch 1E21-N008A.	MSIV 1/2 Isolation on erroneous signal.	Jumpered switch out.
L49994	Outboard MSIV "A".	Limit switch in improper position.	Dropped out RPS K3B relay and would not reset.	Corrected Limit Switch Position.
L50046	1A RHR Heat Exchanger.	Fouled tubes.	Could not obtain desired service water flow.	Cleaned tubes.
L50066	Accident Monitoring Wide Range Level Recorder.	Recorder Out-of-Calibration	Incorrect indication.	Recalibrated.
L50100	RHR Suppression Pool Spray Valve, 1E12-F027B.	Faulty torque switch.	Tripped Thermal Overloads while closing.	Cleaned and adjusted torque switch.
L50121	1A Drywell Pneumatic Compressor.	Bent tubing allowed control air to leak.	Compressor would not load.	Installed new tubing.
L50122	Division I Post-LOCA Oxygen Monitor.	Faulty reagent flow regulator.	Indicated low.	Replaced reagent flow regulator.
L50130	RHR Suppression Pool Spray Valve, 1E12-F027B.	Worn seat and disc.	Excessive leakage through valve B RHR could not be maintained full in standby.	Lapped Valve seat and disc.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L50134	HCU Accumulator for CRD 02-31.	Stem bent on instrument block stop valve.	Leaked nitrogen; potential to cause failure to scram this rod if combined with other events.	Replaced Valve.
L50196	Ammonia Detector.	Optics wire broken during surveillance.	Inoperable detector.	Repaired wire.
L50251	Control Room Venti- lation Air Condition- Compressor "B".	Blown Oil seal	Degraded ventilation from "B" Control Room HVAC.	Replaced seal.
L50416	1B Diesel Generator	Loose bolts on turbocharger	Potential to cause degraded Diesel performance	Torqued bolts.
L50460	1B Diesel Generator K9 Relay.	Bent Plunger Striker on Switchgear 143 cubicle 001.	Prevented proper over-current trip function of ACB 1432.	Installed new plunger striker.
L50481	Standby Liquid Control Pump 1A Discharge Relief Valve.	Would not open at desired pressure.	Potential to cause failure of Standby Liquid Control Piping.	Changed relief setting.
L50525	HCU Accumulator for CRD 34-51.	III Valve leaked Nitrogen.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L50643	HCU for CRD 30-35, and HCU for CRD 46-07.	Instrument Block Valves leaked nitrogen to atmosphere.	Potential to cause failure to scram these rods if combined with other events.	Replaced valves.
L40644	HCU for CRD 54-39.	Instrument block valve leaked nitrogen to atmosphere.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L41909	Reactor Recirculation Discharge Valve, 2B33-FO67A.	Degraded Packing	Packing leak.	Repacked Valve.
L50069	1A RHR Service Water Strainer.	Assembled incorrectly following maintenance.	Strainer leaked considerably; potentially degraded RHR "A" performance.	Reassembled strainer.
L50513	1A RHR Heat Exchanger	Tubes fouled.	Could not obtain required Service Cleaned tubes. Water flow.	
L50336	Safety Relief Valve C.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
LS0337	Safety Relief Valve D.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50338	Safety Relief Valve E.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50339	Safety Relief Valve F.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50342	Safety Relief Valve J.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50344	Safety Relief Valve L.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50345	Safety Relief Valve M.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.



TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L50347	Safety Relief Valve P.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50348	Safety Relief Valve R.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50350	Safety Relief Valve U.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, July 1 through July 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-048-00	6-14-85	"A" RHR WS PRM INOP
85-049-00	6-25-85	Chlorine Detector Trip
85-050-00	6-26-85	"A" VC/VE Ammonia/Chlorine Detector Alarms
85-051-00	6-27-85	Spurious Chlorine Detector Trip.
85-052-00	6-29-85	Manual Reactor Scram
85-053-00	7-17-85	RHR Shutdown Cooling Suction High Flow Isolation Switches Installed Backwards.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373  
UNIT LaSalle One  
DATE August 10, 1985  
COMPLETED BY Richard J. Rohrer  
TELEPHONE (815)357-6761

## OPERATING STATUS

1. REPORTING PERIOD: JULY, 1985 GROSS HOURS IN REPORTING PERIOD: 744  
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY  
(MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078  
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A  
4. REASONS FOR RESTRICTION (IF ANY): N/A
- |   | THIS MONTH    | YR TO DATE      | CUMULATIVE      |
|---|---------------|-----------------|-----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL   | <u>370.8</u>  | <u>3879.5</u>   | <u>10161</u>    |
| 6. REACTOR RESERVE SHUTDOWN HOURS   | <u>373.3</u>  | <u>476.2</u>    | <u>1642</u>     |
| 7. HOURS GENERATOR ON LINE  | <u>351.5</u>  | <u>3726.0</u>   | <u>9783</u>     |
| 8. UNIT RESERVE SHUTDOWN HOURS  | <u>0.0</u>    | <u>0.0</u>      | <u>0.0</u>      |
| 9. GROSS THERMAL ENERGY GENERATED (MWH)   | <u>847778</u> | <u>10290841</u> | <u>27114130</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH)  | <u>266164</u> | <u>3375933</u>  | <u>8846576</u>  |
| 11. NET ELEC. ENERGY GENERATED (MWH)  | <u>249325</u> | <u>3244270</u>  | <u>8439332</u>  |
| 12. REACTOR SERVICE FACTOR  | <u>49.8%</u>  | <u>75.9%</u>    | <u>73.1%</u>    |
| 13. REACTOR AVAILABILITY FACTOR   | <u>100%</u>   | <u>85.2%</u>    | <u>84.9%</u>    |
| 14. UNIT SERVICE FACTOR   | <u>47.2%</u>  | <u>72.9%</u>    | <u>70.4%</u>    |
| 15. UNIT AVAILABILITY FACTOR  | <u>47.2%</u>  | <u>72.9%</u>    | <u>70.4%</u>    |
| 16. UNIT CAPACITY FACTOR (USING MDC)  | <u>32.3%</u>  | <u>61.3%</u>    | <u>58.6%</u>    |
| 17. UNIT CAPACITY FACTOR (USING DESIGN<br>MWe)  | <u>31.1%</u>  | <u>58.9%</u>    | <u>56.3%</u>    |
| 18. UNIT FORCED OUTAGE RATE   | <u>52.6</u>   | <u>23.6%</u>    | <u>19.9%</u>    |
| 19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)<br>Unit one is scheduled for a refueling, maintenance, modification, and<br>surveillance outage beginning September 3, 1985 and lasting 26 weeks. |               |                 |                 |
| 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: <u>NA</u>  |               |                 |                 |

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373

UNIT: LASALLE ONE

DATE: August 10, 1985

COMPLETED BY: Richard J. Rohrer

TELEPHONE: (815) 357-6761

MONTH: JULY, 1985

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1. 170	17. -18
2. 439	18. -17
3. 814	19. -17
4. 749	20. -18
5. 884	21. -18
6. 871	22. -21
7. 785	23. -17
8. 878	24. -15
9. 877	25. -16
10. 879	26. -16
11. 821	27. -16
12. 51	28. 90
13. -18	29. 695
14. -17	30. 724
15. -17	31. 923
16. -19	

## ATTACHMENT E

## 3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373UNIT NAME LaSalle OneDATE JULY 10, 1985COMPLETED BY Richard J. RohrerTELEPHONE (815)357-6761REPORT MONTH JULY 1985

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
14	850629	F	2.5	A	4	Continuation of outage from previous month.
15	850712	F	390.0	A	2	Unit shutdown due to inoperable suppression pool spray valve 1E12-F027B.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO &amp; TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no Safety Relief Valves Operated for Unit One during this reporting period.

## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-552-85	1C RHR Pump	Lubrication
1-554-85	1A RHR Heat Exchanger	Clean Tubes
1-556-85	1E12-F027B	Repair Valve
1-557-85	1E12-F027B	Repair Valve
1-558-85	1E12-F027B	Adjust Limit Switches
1-565-85	1E12-F063A	Keep RHR pressurized
1-568-85	1B RHR Service Water Pump	Repair Pump
1-572-85	1B D/G	Lubrication
1-573-85	Shutdown Cooling Suction Header	Prevent Initiation with vents open.
1-574-85	1E12-F027B	Maintain Primary Containment.
1-576-85	1B D/G	Calibration
1-577-85	1E12-F027B	Maintain Primary Containment
1-578-85	1E12-F027B	Remove Actuator
1-579-85	1E12-F027B	Remove Actuator
1-585-85	1E12-F053A	Repair Valve
1-586-85	1E12-F053A	Repair Actuator
1-587-85	1E12-F004A	Prevent Operation
1-588-85	1E12-F027B	Repair Valve.
1-594-85	1B RHR Pump	Oil Sample
1-609-85	1E12-F004A	Repair Torque Switch



<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-611-85	1E12-F023	Prevent operation
1-626-85	1E12-N012AA	Repipe Instrument
1-630-85	1E12-F004A	Repair Limitorque
1-636-85	1A RHR Heat Exchanger	Clean Tubes
1-638-85	1E12-D300A	Inspect and Clean
1-642-85	1E12-F023	Verify wiring on valve
1-652-85	1E12-F008	Perform LIS-NB-311
1-653-85	1E12-D300A	Repair Leaks
1-655-85	RHR Shutdown Cooling Valves	Vent path for LST-85-45
1-658-85	1E12-F336A	Replace Retainer Ring

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculation Manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JULY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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    - 3. Off-Site Dose Calculation Manual Changes
    - 4. Major Changes to Radioactive Waste Treatment System

## I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984, and commercial power operation was commenced on June 19, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

JULY 1-19

July 1, 0001 Hours      Reactor subcritical. Unit two still in scheduled outage.

JULY 20-31

July 20, 2125 Hours      Reactor Critical

July 22, 0530 Hours      Generator Synchronized to Grid

July 22, 0630 Hours      Main Turbine Trip Due to High Level in MSR Drain Tank.

July 22, 0800 Hours      Generator Synchronized to grid.

July 23, 0300 Hours      Removed Main Turbine From Grid for RCIC Surveillance

July 23, 0420 Hours      Generator Synchronized to Grid

July 24, 0700 Hours      Reactor Power at 34%

July 25, 1500 Hours      Reactor Power at 64%

July 26, 1500 Hours      Reactor Power at 82%.

July 27, 0700 Hours      Reactor Power reduced to 67% for Rod Shuffle

July 27, 1500 Hours      Reactor Power at 86%.

July 31, 0700 Hours      Reactor Power at 96%.

July 31, 1500 Hours      Reactor Power at 78%.

July 31, 2300 Hours      Reactor Power Reduced to 23%. Bringing Unit Down to Investigate High Drywell Temperatures. The Reactor was Critical for 266 Hours and 35 Minutes During the Month of July.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

There were no Amendments to the facility License or Technical Specifications for this reporting Month.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L43822	HCU for CRD 02-39	Scram pilot valve had an air leak.	Potential to cause half-scam for this rod.	Replaced O-ring seals in valve.
L42825	HCU for CRD 02-39	Scram pilot valve had an air leak.	Potential to cause half-scam for this rod.	Replaced O-ring seals in valve.
L46594	HCU for CRD 10-31.	Scram pilot valve had an air leak.	Potential to cause half-scam for this rod.	Rebuilt valve.
L46655	RCIC Outboard Steam Isolation Valve.	Degraded valve packing.	Significant steam leak through packing.	Repacked valve.
L47320	Outboard Feedwater Check Valve "B".	Actuating Cylinder leaked air.	Degraded Valve Operation.	Rebuilt actuating cylinder.
L47356	Outboard Feedwater Check Valve "A".	Both actuating cylinders leaked air.	Degraded Valve Operation.	Rebuilt Actuating cylinders
L47518	Drywell Pneumatic Dryer Purge Valve.	Worn Valve seat.	Leakage in excess of desired amount.	Lapped seat.
L47529	Floor Drain Inboard Isolation Valve.	Crud obstructing valve motion.	Valve leaked in excess of allowable amount.	Cleaned valve.
L47591	Safety Relief Valve "E".	Damaged Valve Seat and disc.	Valve leaked steam by.	Replaced valve.
L47592	Safety Relief Valve "R".	Damaged Valve seat and disc.	Valve leaked steam by.	Replaced valve.
L47631	RCIC Turbine Exhaust Isolation valve, 2E51-F068.	Limit Switches out of Adjustment.	Valve would not fully close except manually.	Repositioned limit switches.



TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L47692	HCU for CRD 18-07.	Scram, pilot valve had a severe air leak.	Potential to cause a half-scam of this control rod.	Rebuilt valve.
L47763	HCU for CRD 06-27.	Scram pilot valve leaked air.	Potential to cause a half-scam of this control rod.	Rebuilt valve.
L48300	Hydrogen Recombiner exhaust upstream isolation valve, 2HG006A.	Worn valve disc.	Failed local leak rate test.	Replaced Valve disc.
L48693	Reactor Pressure interlock switch, 2B21-N039L.	No instrument rack stop valve.	Could not isolate instrument.	Installed new stop valve.
L49227	Hydrogen Recombiner exhaust downstream isolation valve, 2HG005A.	Worn Valve disc. and seat.	Valve failed local leak rate test.	Replaced disc and lapped seat.
L49768	RHR Shutdown Cooling High Flow isolation switches, 2E12-N012AA and AB.	Switches were piped backwards.	Switches were inoperable.	Piping corrected.
L49788	HCU for CRD 06-35.	Leaking drain valve on accumulator.	Potential to cause failure to scram this rod if combined with other events.	Rebuilt Valve.
L49843	Feedwater Check Valve 2B21-F032A.	Incorrect solenoids installed.	Air leaked on actuator; Possibly possibly degraded valve operation.	Installed correct solenoids.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L49856	Scram Reset Switch.	Switch stuck in reset position.	Groups 1 and 4 would automatically reset if scram signal cleared.	Installed new switch operator.
L49858	Safety Relief Valve A.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49860	Safety Relief Valve C.	Broken Set Screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49861	Safety Relief Valve D.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49863	Safety Relief Valve F.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49864	Safety Relief Valve G.	Broken set screw in nozzle ring.	No significant effect. set screw.	Replaced with improved
L49865	Safety Relief Valve H.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49868	Safety Relief Valve L.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49869	Safety Relief Valve M.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49871	Safety Relief Valve P.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49875	Safety Relief Valve V.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.

TABLE 1

CORRECTIVE MAINTENANCE OF  
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L49900	Diesel Generator Cooling Water Pump B.	Bent Actuating Arm on auxiliary contacts.	Cooling Water Pump did not start when HPCS Pump started.	Adjusted Actuating arm.
L49983	Various HCU's for Control Rod Drives.	Instrument block stop valves leaked at steam.	Potential to cause failure to scram the affected rods if combined with other events.	Replaced leaking valves.
L49988	HCU for CRD 26-03.	Instrument block stop valve leaked by stem.	Potential to cause failure to scram this rod if combined with other events.	Replace Valve.
LS0366	Division III Battery Charger.	High Voltage shutdown board set incorrectly.	Charger output would not reach desired value.	Reset high voltage shutdown board.
L50421	2A Diesel Generator.	Loose bolts on turbocharger.	Potential for degraded diesel operation.	Torqued bolts.
L50479	HCU for CRD 18-39.	Instrument block stop valve had a severe stem leak.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L50514	HCU for CRD 50-31.	Instrument block stop valve had a stem leak.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L50602	RCIC Turbine.	Governor out of adjustment.	Could not control turbine speed or pump output.	Adjusted governor.
L50636	HCU for CRD 58-31.	Instrument Block stop valve leaked at packing.	Potential to cuase failure to scram this rod if combined with other events.	Replaced valve.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, July 1 through July 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-029-00	6-10-85	Pressure Switch 2B21-N037AA and 2B21-N037AB piped Backwards.
85-030-00	6-26-85	Group II Isolation
85-031-00	6-22-85	RHR Shutdown Cooling High Suction Flow Isolation
85-032-00	7-1-85	Leak Detection Div. I & II RHR $\Delta T$
85-033-00	7-1-85	RHR Shutdown Cooling Isolation.
85-034-00	6-25-85	Temporary Voltage Degradation During 237 Transformer Failure.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374  
 UNIT LaSalle Two  
 DATE August 10, 1985  
 COMPLETED BY Richard J. Rohrer  
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: July, 1985 GROSS HOURS IN REPORTING PERIOD: 744
  2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
  3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
  4. REASONS FOR RESTRICTION (IF ANY): N/A
- |  | THIS MONTH    | YR TO DATE     | CUMULATIVE     |
|--|---------------|----------------|----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL    | <u>266.6</u>  | <u>1666.4</u>  | <u>3278.2</u>  |
| 6. REACTOR RESERVE SHUTDOWN HOURS          | <u>0.0</u>    | <u>0.0</u>     | <u>125.3</u>   |
| 7. HOURS GENERATOR ON LINE                 | <u>231.7</u>  | <u>1629.0</u>  | <u>3166.4</u>  |
| 8. UNIT RESERVE SHUTDOWN HOURS             | <u>0.0</u>    | <u>0.0</u>     | <u>0.0</u>     |
| 9. GROSS THERMAL ENERGY GENERATED (MWH)    | <u>493368</u> | <u>4880753</u> | <u>9388345</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH)     | <u>156348</u> | <u>1616735</u> | <u>3101721</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH)       | <u>141584</u> | <u>1515220</u> | <u>2907537</u> |
| 12. REACTOR SERVICE FACTOR                 | <u>35.8%</u>  | <u>32.6%</u>   | <u>47.6%</u>   |
| 13. REACTOR AVAILABILITY FACTOR            | <u>35.8%</u>  | <u>32.6%</u>   | <u>49.4%</u>   |
| 14. UNIT SERVICE FACTOR                    | <u>31.3%</u>  | <u>31.9%</u>   | <u>46.0%</u>   |
| 15. UNIT AVAILABILITY FACTOR               | <u>31.1%</u>  | <u>31.9%</u>   | <u>46.0%</u>   |
| 16. UNIT CAPACITY FACTOR (USING MDC)       | <u>18.4%</u>  | <u>28.6%</u>   | <u>40.8%</u>   |
| 17. UNIT CAPACITY FACTOR(USING DESIGN MWe) | <u>17.7%</u>  | <u>27.5%</u>   | <u>39.2%</u>   |
| 18. UNIT FORCED OUTAGE RATE                | <u>0.0%</u>   | <u>0.0%</u>    | <u>7.0%</u>    |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):  
N/A
  20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374

UNIT: LASALLE TWO

DATE: August 10, 1985

COMPLETED BY: Richard J. Rohrer

TELEPHONE: (815) 357-6761

MONTH: July 1985

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	-13
2.	-13
3.	-15
4.	-18
5.	-14
6.	-15
7.	-15
8.	-15
9.	-14
10.	-15
11.	-18
12.	-16
13.	-17
14.	-17
15.	-17
16.	-18

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	-17
18.	-17
19.	-16
20.	-17
21.	-18
22.	80
23.	165
24.	300
25.	563
26.	807
27.	668
28.	869
29.	1003
30.	1016
31.	765

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374

UNIT NAME LaSalle Two

DATE August 10, 1985

COMPLETED BY Richard J. Rohrer

TELEPHONE (815)357-6761

REPORT MONTH JUNE 1985

NO.	DATE	TYPE	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED S: SCHEDULED				
3	850228	S	509.5	B	4	Maintenance and Surveillance outage continued from February.
4	850722	F	0.0	A	5	Turbine trip due to high level in MSR Drain Tank.
5	850723	S	0.0	B	5	Took Turbine off for RCIC Surveillance.



E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO &amp; TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
7-23-85	2B21-F013E	2 Manual	960 PSIG	Inadvertantly Opened during Set Pressure Verification Test.
7-23-85	2B21-F013N	1 Manual	960 PSIG	Inadvertantly Opened during Set Pressure Verification Test.

## 2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-1013-85	LPCS Water Leg Pump	Lubrication
2-1019-85	2E12-F024A	Repair Limitorque
2-1023-85	2E12-F024A	Disconnect Motor
2-1030-85	2A D/G	Lubrication
2-1059-85	A/B RHR Service Water Pumps	Polarization Test
2-1062-85	A/B RHR Shutdown Cooling	LIS-NB-211
2-1063-85	C/D RHR Service Water Pump	Surveillance

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculation manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the radioactive waste treatment system during this reporting period.



**Commonwealth Edison**  
LaSalle County Nuclear Station  
Rural Route #1, Box 220  
Marseilles, Illinois 61341  
Telephone 815/357-6761

August 10, 1985

Director, Office of Management Information  
and Program Control  
United States Nuclear Regulatory Commission  
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering  
LaSalle County Nuclear Power Station for the period July 1 through July 31,  
1985.

Very truly yours,

*G. J. Diederich*  
G. J. Diederich  
Station Manager  
LaSalle County Station

GJD/RJR/crh

Enclosure

xc: J. G. Keppler, NRC, Region III  
NRC Resident Inspector LaSalle  
Gary Wright, Ill. Dept. of Nuclear Safety  
D. P. Galle, CEC  
D. L. Farrar, CEC  
INPO Records Center  
Ron A. Johnson, PIP Coordinator SNED  
J. E. Ellis, GE Resident  
J. M. Nowicki, Asst. Comptroller  
H. E. Bliss, Nuclear Fuel Services Manager  
C. F. Dillon, Senior Financial Coordinator, LaSalle

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