

LICENSEE EVENT REPORT (LER)

APPROVED ONS NO. 2180-0104
EXPIRES 8/31/83

FACILITY NAME (1)

Limerick Generating Station - Unit 1

DOCKET NUMBER (2)

0 5 0 0 0 3 5 2 1 OF 0 3

PAGE (3)

TITLE (4)

Engineered Safety Feature Actuation - RWCU Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME		DOCKET NUMBER (8)	
0	8	24	85	0	71	0	9	1			0 5 0 0 0 1 1	
DEACTATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
2			20.402(a)			20.406(a)			X 00.730(1)(i)(H)			73.710(i)
POWER LEVEL (10)			20.406(b)(1)(i)			00.730(1)(i)			00.730(1)(i)(H)			73.710(i)
0 2 3			20.406(b)(2)(i)(H)			00.730(1)(i)			00.730(1)(i)(H)			OTHER (Specify in Abstract below and in Text, NRC Form 300A)
			20.406(b)(3)(i)(H)			00.730(1)(i)			00.730(1)(i)(H)(A)			
			20.406(b)(4)(i)(H)			00.730(1)(i)			00.730(1)(i)(H)(B)			
			20.406(b)(5)(i)(H)			00.730(1)(i)			00.730(1)(i)(H)			

LICENSEE CONTACT FOR THIS LER (12)

NAME

John C. Nagle, Senior Engineer, Licensing Section

TELEPHONE NUMBER

2 1 5 8 4 1 - 5 1 8 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input checked="" type="checkbox"/>	<input type="checkbox"/>				

ABSTRACT (Summarize in 1000 spaces, i.e., approximately fifteen single-spaced typewritten lines) (16)

Abstract: 85-071

On August 24, 1985 at 0219 with Unit 1 at 23 percent power, the Reactor Water Cleanup (RWCU) system became isolated when the inboard isolation valve received a spurious signal from the Differential Temperature Switch TDTS-44-1N602E. An Instrument and Control (I&C) technician was installing test equipment on a nearby temperature switch when the isolation occurred. The investigation found the isolation to be duplicated by touching the front of the switch module. A loose connection was determined to be the cause of the isolation signal. The temperature switch connector was cleaned and tightened. The RWCU system was returned to service by 0340.

The EIIS code for the affected system is CE and the EIIS code for the defective component is TS.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2 8 5	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	0 7 1	0 0	0 2	OF	0 3

TEXT (If more space is required, use additional NRC Form 368a) (17)

Description of the Event:

On August 24, 1985 at 0219 with Unit 1 at 23 percent power, a differential temperature switch (TDTS-44-1N602E, Riley Company Model 86) generated an isolation signal causing an Engineered Safety Feature (ESF) closure of an inboard RWCU isolation valve (HV-44-1F001). An I&C technician was installing test equipment on a nearby temperature switch when the isolation occurred. An investigation determined the cause of the isolation signal to be a loose connection on the switch. The RWCU system was returned to service by 0340.

Consequences of the Event:

The RWCU system was being used to maintain reactor water chemistry before the isolation occurred. The RWCU system was returned to service in 81 minutes. There were no adverse consequences as a result of this event, since reactor water chemistry remained relatively stable.

Cause of the Event:

A loose connection on the edge of the differential temperature switch was identified as the cause for the RWCU system isolation. During the investigation, the isolation signal was reproduced by touching the front of the switch module.

Corrective Actions:

The differential temperature switch module was removed from service and its connector was cleaned. The component and associated isolations were then functionally tested and returned to service.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/86

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			YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			8 5	0 7 1	0 0	0 1 3	OF	1 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Previous Similar Occurrences:

The following LERs reported RWCU system isolations which were caused by defective Riley temperature modules:

84-012	85-001
84-026	85-027
84-034	85-055
84-036	85-061

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September 19, 1985

Docket No. 50-352

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Washington, DC 20555

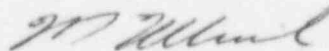
SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This LER deals with an engineered safety feature actuation of a Reactor Water Cleanup isolation valve.

Reference:	Docket No. 50-352
Report Number:	85-071
Revision Number:	00
Event Date:	August 24, 1985
Report Date:	September 19, 1985
Facility:	Limerick Generating Station P.O. Box A, Sanatoga, PA 19464

This LER is being submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
E. M. Kelly, Senior Resident Site Inspector
See Service List

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